

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

<u>/O/I/</u>	<u>/V/A/N/A/S/2/</u> (2)	<u>/O/O/-/O/O/O/O/O/-/O/O/</u> (3)	<u>/4/1/1/1/1/</u> (4)	<u>/ / /</u> (5)
LICENSEE CODE		LICENCE NUMBER	LICENSE TYPE	CAT

<u>/0/1/</u>	REPORT	<u>/L/</u> (6)	<u>/0/5/0/0/0/3/3/9/</u> (7)	<u>/0/4/1/9/8/3/</u> (8)	<u>0/5/1/1/8/3/</u> (9)
	SOURCE		<u>SECRET NUMBER</u>	<u>EVENT DATE</u>	<u>REPORT DATE</u>

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On April 19, 1983, with Unit 2 in the Refueling Mode, Fire Door S71-18, between /
 /0/3/ / the 7H Emergency Diesel Room and the Turbine Building, would not latch. Since a /
 /0/4/ / fire watch was immediately posted the public health and safety were not affected. /
 /0/5/ / The degradation of fire doors between the Emergency Diesel Rooms and Turbine /
 /0/6/ / Building due to differential pressure between the areas during diesel operation /
 /0/7/ / is a recurring event. This event is contrary to T.S. 3.7.15 and reportable pur- /
 /0/8/ / suant to T.S. 6.9.1.9.b. /

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMP. COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
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<u>/O/9/</u>	<u>/A/B/</u>	<u>(11)</u>	<u>/E/</u>	<u>(12)</u>	<u>/B/</u>	<u>(13)</u>	<u>/X/X/X/X/X/X/</u>	<u>(14)</u>	<u>/Z/</u>	<u>(15)</u>	<u>/Z/</u>	<u>(16)</u>
					SEQUENTIAL			OCCURRENCE		REPORT		REVISION
	LER/RO	EVENT	YEAR		REPORT NO.			CODE		TYPE		NO.
(17)	REPORT											
	NUMBER	/8/3/			/-/ 0/3/4/			/0/3/		/L/		/0/

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
/B/ (18)	/X/ (19)	/Z/ (20)	/Z/ (21)	/0/0/0/0/ (22)	/N/ (23)	/N/ (24)	/A/ (25)	/C/1/7/5/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / Fire Door S71-18 would not latch because the latch was sticking. The latch was /
 /1/1/ / lubricated and operation of the fire door tested satisfactorily. Design modifi- /
 /1/2/ / cation to the fire doors between the Diesel Rooms and the Turbine Building are /
 /1/3/ / being pursued. /
 /1/4/ /

FACILITY		METHOD OF		
STATUS	ZPOWER	OTHER STATUS	DISCOVERY	DISCOVERY DESCRIPTION (32)
/H/ (28)	/0/0/0/ (29)	/ NA / (30)	/A/ (31)	/ Operator Observation /

ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
/1/6/	/Z/ (33)	/Z/ (34)	NA

PERSONNEL EXPOSURES		
NUMBER	TYPE	DESCRIPTION (39)
/1/7/	/0/0/0/ (37) /2/ (38)	/ NA

[illegible]

DATE	TIME	LOCATION	DESCRIPTION	REMARKS
1/18/	10/00/	(40)	NA	8312300006 831018
	LOSS OF	OR DAMAGE TO FACILITY		PDR FOIA
	TYPE	DESCRIPTION	(43)	GRABER83-562 PDR

<u>/1/9/</u>	<u>/Z/</u>	(42)	/	NA
PUBLICITY				

ISSUED		DESCRIPTION (45)	NRC USE ONLY
/2/0/	/N/ (44)	/ NA	/ / / / / / / / / / / / / / /

NAME OF PREPARER W. R. CARTWRIGHT PHONE (703) 894-5151

December 20, 1982

JBW
JPC

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Maria Lane, Suite 210
Walnut Creek, California 94596-5368

LER
LER # 361-82152
EVENT DATE 821131
INFO RCVD DATE 821230 SP
RCVD DATE

Attention: Mr. R. H. Engelken, Regional Administrator

Dear Sir:

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 82-152
San Onofre Nuclear Generating Station, Unit 2

This submittal is in accordance with the reporting requirements of Section 6.9.1.13.b of Appendix A to Facility Operating License NPF-10. It describes a reportable occurrence involving Limiting Condition for Operation (LCO) 3.3.3.6 associated with Accident Monitoring Instrumentation. A completed copy of LER 82-152 is enclosed.

While in Mode 3, at 2010 on November 21, 1982, the daily surveillance testing in accordance with procedure 5023-3-3.21.1 was in progress. Due to loss of process flow, as indicated by radiation indicator 2RI-7872-1, the Condenser Evacuation System radiation monitor (wide range) was declared inoperable. Action Statement 22 of Table 3.3-10 associated with this inoperable monitor results in an undefined action requirement due to a typographical error in the Technical Specifications. Correction of this Action Statement has been requested in License Amendment Application 15 and the action taken was as defined in that application (i.e. either restore the inoperable monitor to operable status within 7 days, or be in hot shutdown within the next 12 hours).

Subsequent investigation revealed that the monitor problem was attributable to a faulty velocity probe associated with the monitor. The faulty probe was replaced and the monitor declared operable in accordance with 5023-3-3.21.1 at 1705 on November 24, 1982. Discussions with the probe manufacturer indicate that this failure is infrequent and, therefore, this is considered an isolated event. No further corrective action is warranted at this time.

R. H. Engelken

-2-

December 20, 1982

There was no impact on plant operations or the health and safety of plant personnel or the public as a result of this event.

If there are any questions regarding the above, please contact me.

Sincerely,

HBR/TVM

H. B. RAY
STATION MANAGER

LIMayweather:VM

Enclosure: LER 82-152

cc: A. E. Chaffee (USNRC Resident Inspector, San Onofre Unit 2)

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement

U. S. Nuclear Regulatory Commission
Office of Management Information and Program Control

Institute of Nuclear Power Operations