



Westinghouse
Electric Corporation

Water Reactor
Divisions

Nuclear Technology Division

Box 355
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December 19, 1983

NS-EPR-2867

Ref: a) NS-EPR-2757

Mr. Richard C. Vollmer, Director
Office of Engineering
Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, MD 20014

Subject: Aging of Cast Stainless Steel Piping

Dear Mr. Vollmer:

Over the past several months Westinghouse and a number of utilities have provided the Staff with information to justify the elimination of postulated reactor coolant system (RCS) primary loop pipe breaks both on a generic (Reference a) and a specific plant basis. In addition, Westinghouse, utilities and the Staff have discussed the technical methodology associated with the elimination of pipe breaks on several occasions. At such a meeting on October 27, 1983, Westinghouse and the Staff discussed the effect of aging on the material properties of cast stainless steel. As a result of this meeting, the Staff requested that Westinghouse provide information on this subject.

Based on the Staff request Westinghouse is providing the following information:

1. Ten (10) copies of Topical Report WCAP 10456, "Effects of Thermal Aging on the Structural Integrity of Cast Stainless Steel Piping for the Westinghouse Nuclear Steam Supply System," December, 1983, proprietary.
2. Ten (10) copies of Topical Report WCAP 10457, "The Effects of Thermal Aging on the Structural Integrity of Cast Stainless Steel Piping for the Westinghouse Nuclear Steam Supply System," December, 1983, non-proprietary.

Also enclosed is one (1) Application for Withholding, AW-83-112.

The enclosed reports provide information on the effect of aging on the material properties of cast stainless steel piping. Additionally, these reports demonstrate that although the material properties of cast stainless steel piping

8312290254 831219
PDR TOPRP EMVWEST
B PDR

YCOI 1/10 Prop
Change: 1/10 Non Prop
PDR
NSIC
NTIS } *Non Prop*

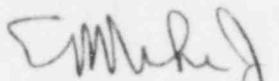
are affected by aging, margins still exist in the acceptance criteria used to justify the elimination of pipe breaks in piping systems of cast stainless steel. The enclosed reports will be referenced by utilities on specific plant applications where appropriate. When referenced on a specific plant docket, the applicability of the enclosed reports to that docket will also be demonstrated.

This submittal contains proprietary information of Westinghouse Electric Corporation. In conformance with the requirements of 10CFR2.790, as amended of the Commission's regulations, we are enclosing with this submittal an application for withholding from public disclosure and an affidavit. The affidavit sets forth the basis for which the information may be withheld from public disclosure by the Commission.

Correspondence with respect to the affidavit or application for withholding should reference AW-83-112 and should be addressed to R. A. Wiesemann, Manager, Regulatory and Legislative Affairs, Westinghouse Electric Corporation, P.O. Box 355, Pittsburgh, PA 15230.

Very truly yours,

WESTINGHOUSE ELECTRIC CORPORATION



E. P. Rahe, Jr., Manager
Nuclear Safety Department

JJM/anj

Enclosures

cc: K. Wickman - NRC
R. Klecker - NRC
K. Jabbour - NRC
R. Bosnak - NRC
H. Schmidt - TUSI
R. Widler - Duke Power
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