APPROVED BY OMB .. U.S. NUCLEAR REGULATORY COMMISSION 3150-0011 NAC FORM 366 (12-81) 10 CFR 50 LICENSEE EVENT REPORT (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK 0 - 0 0 0 0 - 0 0 3 4 1 1 1 1 1 (2) 0 BRF 0 1 CON'T 9 7 1 2 1 0 8 3 8 1 2 2 0 8 40 EVENT DATE 74 75 REPORT DATE 3 (9) REPORT L 6 0 5 0 0 2 5 011 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) | On 12/10/83, with unit 1 in a refueling outage, an operator observed RHR 0 2 pump 1C motor was sparking and smoking. The motor was shut down and subsequently 0 3 restarted. After restarting, the motor tripped on overcurrent. With RHR Loop 0 4 II out-of-service, only RHR pump 1A, Loop I, was operable. This exceeded limit 0 5 of LCO, Tech Spec 3.5.B.9. Limits were exceeded for two hours. There was no 0 6 effect on public health and safety. Reactor vessel temperature remained within 0 7 limits required by Tech Specs 3.6.A.3 and .5. 0 8 COMP. SUBCODE SUBCODE CAUSE CAUSE COMPONENT CODE Z (13) (12) M TIO X 0 9 REVISION CCURRENCE SEQUENTIAL REPORT NO. CODE 0 0 17 HEPORT 0 6 8 8 3 2.8 29 COMPONENT 26 PRIME COMP. CHMENT SHUTDOWN FUTURE TION METHOD ACTION N (24) N (25) GIQ (23) (21) 0 0 C (18) X (19) Z (20) CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The GE model number 5K6348XC23A, 2000 HP, 3 phase motor is believed to have 10 tripped due to winding failure. RHR Loop II was placed back in-service and 1 RHR pump 1C motor will be replaced. After failure evaluation the failure mode and recurrence control will be addressed in a follow-up report to this LER by 6/8/84. 4 METHOD OF DISCOVERY DISCOVERY DESCRIPTION FACILITY (30) OTHER STATUS S POWER Operator observation A (31) H 28 0 0 0 0 29 NA 1 5 13 12 10 CONTENT OF RELEASE (36) LOCATION OF RELEASE AMOUNT OF ACTIVITY Z 3 Z 3 NA 1 6 80 10 PERFONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39) 0 0 0 0 37 Z 38 NA 80 12 PERSONNEL INJURIES NUMBER 0 0 0 0 NA 8 TF22 8312280320 831220 PDR ADDCK 05000259 LOSS OF OR DAMAGE TO FACILITY Z (42) 9 S NAC USE ONLY PUBLICITY ISSUED DESCRIPTION 45 NA N (44) 2 0 PHONE 205-729-0889 Walter T. Christopher NAME OF PREPARER

Tennessee Valley Authority Browns Ferry Nuclear Plant

Form BF 17 BF 15.2 2/19/82

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 83068 Technical Specification Involved 3.5.8.9

Reported Under Technical Specification 6.7.2.a(2) * Date Due NRC 12/24/83

Event Narrative:

Unit 1 was in a refueling outage with irradiated fuel in the vessel and the vessel at atmospheric pressure; unit 2 was operating normally at 100-percent power; unit 3 was in a refueling outage. Unit 1 was the only unit affected by this event. On December 10, 1983, an operator observed that RHR pump 1C motor was sparking and smoking. The motor was shut down and subsequently restarted. After being restarted and running for a few minutes the motor tripped on overcurrent at 2045 hours. A and C phase and ground instantaneous overcurrent relays actuated. RHR Loop II was out-of-service. Therefore, RHR pump 1A, Loop I, was the only RHR pump operable. This exceeded the limiting condition of operation as defined by Technical Specification (T.S.) 3.5.B.9. Allunits 1 and 2 diesel generators were operable. RHR Loop II was returned to service at 2243 hours. T.S. limits 3.5.B.9 were therefore exceeded for 2 hours. RHR pump 1C motor is being replaced. The GE model number 5K6348XC23A, 2000 HP, 3 phase motor is believed to have tripped due to winding failure. There was no effect on public health and safety. Reactor vessel temperature was maintained within limits as required by T.S. 3.6.A.3 and .5 during this event. Following a failure evaluation the motor failure mode and recurrence control will be addressed in a follow-up report to this LER by June 1, 1984.

* Previous Similar Events:

BFRO-50-296/81034

*Revision:

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

December 20, 1983

03 DEC 22 A11: 10

Mr. James P. O'Reilly, Director U.S. Nuclear Regulatory Commission Suite 2900 101 Marietta Street, NW. Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 - DOCKET NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE REPORT BFR0-50-259/83068

The enclosed report provides details concerning failure of a residual heat removal system pump motor. This report is submitted in accordance with Browns Ferry unit 1 Technical Specification 6.7.2.a(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

Director of Nuclear Power

Enclosure cc (Enclosure): Document Control Desk U.S. Nuclear Regulatory Commission Washington, D.C. 20555

> Records Center Institute of Nuclear Power Operations Suite 1500 1100 Circle 75 Parkway Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

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