Project No. 669

Mr. E. E. Kintner, Chairman Advanced Light Water Reactor Steering Committee GPU Nuclear Corporation 100 Interpace Parkway Parsippany, New Jersey 07054

Dear Mr. Kintner:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION ON EPRI ADVANCED LIGHT WATER REACTOR (ALWR) REQUIREMENTS DOCUMENT FOR PASSIVE PLANT DESIGNS + OFFICE OF NUCLEAR REGULATORY RESEARCH (TAC NO. 77862)

As a result of its review of Volume III of the EPRI ALWR Requirements Document, submitted by letter dated September 7, 1997, the staff has determined that it needs additional information in order to complete its review of the design criteria. The additional information is needed in areas addressed during the Office of Nuclear Reactor Research's raview of Appendix B to Chapter 1, as discussed in the enclosure to this letter.

The reporting and/or recording requirements contained in this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

Please respond to this request within 60 days of the date of receipt of this letter. If you have any questions regarding this matter please contact the project manager, J. Wilson, at (301) 492-1118.

> Sincerely. James H. Wilson, Project Manager Standardization Project Directorate Division of Advanced Reactors and Special Projects Office of Nuclear Reactor Regulation

Enclosure: As stated

cc w/enclosure: See next page

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DATE :09/1 /91 :09/11 /91/1 Mr. E. E. Kintner, Chairman ALWR Utility Steering Committee

Project No. 669 EPRI

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## REQUEST FOR ADDITIONAL INFORMATION EPRI ALMR REQUIREMENTS DOCUMENT FOR PASSIVE PLANT DESIGNS OFFICE OF NUCLEAR REGULATORY RESEARCH

EPRI listed all generic issues whose resolution was considered in the design of the passive ALWR in Table B.3-1 of Appendix B. In addition, a footnote to that table lists other generic issues that EPRI "...determined, through the NUREG-1197 selection criteria, to be not technically relevant to the ALWR design..."

Based upon review of that table and footnote, the staff has identified the following additional information that is needed in order to proceed with its review.

- 1. A number of issues are listed in Table B.3-1 and discussed in Section 3 of Appendix B. For each of the three subsets of these issues described below, EPRI should provide a cross-index by individual issue, indicating specific additional sections of the Requirements Document that are directly related to the resolution of the issue in the passive design.
- a. Those issues where EPRI's discussion for the passive plant is the same as for the evolutionary plant include the following:

GI C-8 MSL Leakage Control
GI-15 Radiation Effects on RV Supports
GI-29 Boli Degradation
GI-57 Fire System Inadvert Actuation
GI-75 Generic Implications of ATWS at Salem
GI-79 Unanalyzed RV for Natural Cooldown Cycles

GI-113

Those issues where EPRI's discussion for the passive design is substantially different from evolutionary design include the following:

Dynamic Testing of Large Hydraulic Snubbers

GI P-17 Criteria for Safety Related Operator Actions G1-70 PORV & BV Reliability G1-84 PORVs on CE Plants G1-87 HPCI Turbine SL Break w/o Isolation GI-94 More LTOP Protection GI-105 Inter-System LOCA Hydrogen Control in Large Dry Containment G1-121 GI-128 Electrical Power Issues GI-130 SW Failure at Multi-Plant Sites HF 5.1 Local Control Stations Review Criteria for Human Factors Aspects of ADV Controls HF 5.2 and Instrumentation

c. EPRI's discussion of some issues has not yet been reviewed by the staff for the evolutionary or the passive plant, because the material was not cited as being relevant to the issue in the evolutionary Requirements Document (i.e., no cross-index was provided for the evolutionary plant similar to that being requested for the passive plant). For these issues, the cross-index of issues with Requirements Document sections should be provided for both the passive and the evolutionary plants. These include:

GI-83 Control Room Habitability
GI-1.D.3 Safety System Status and Monitoring

2. A number of issues listed in the footnote of Table B.3-1 were considered by the staff during review of the evolutionary Requirements Document. However, some issues in the passive plant were "...determined, through the NUREG-1197 selection criteria, to be not technically relevant to the ALWR design...." No further discussion of these issues is presented in Section 3 of Appendix B of the Passive Plant Requirements Document. EPRI should document the reasons for such "selection for non-consideration." or at minimum should reference within Appendix B, Section 3, specific other sections and pages where such documentation can be found. These issues include:

GI-23 RC Pump Seals
GI B-56 Diesel Generator Reliability
GI-135 SG & SL Overfill
GI B-55 Target Rock S/R Valves

GI-106 Piping Inflammable Gases HF 4.4 Guides for Upgrading Other Procedures

3. A number of issues considered by the staff for the evolutionary plant are neither discussed nor listed as "irrelevant" for the passive plant. EPRI should provide a discussion of resolution of these issues for the passive plant design, or should provide a discussion explaining their irrelevance to the design. These include:

USI A-17 System Interactions

USI A-29 Sabotage

USI A-40 Seismic Criteria USI A-44 Station Blackout

USI A-45 Decay Heat Removal

USI A-46 Seismic Qualification

US1 A-47 Safety Implications of Control Systems

USI A-48 Hydrogen Control in Ice & MK-III Containments

USI A-49 PTS

GI B-5 Buckling of Steel Containments

GI B-29 Ultimate Heat Sinks

GI-B-32 Ice Effects

GI D-2 GI-2	ECCS Improvements Failure of Protection Devices
GI-32	Flow Block (clams)
GI-43	Air System Reliability
GI-51	Reliability of Open Cycle Cooling System
GI-76	instrumentation and Control Power Interactions
G1-82	eyond DBAs in Spent Fuel Pools
GI-91	Delaval Diesel Genenerator Main Crank Failure
G1-93	Steam Binding of Auxiliary FW Pumps
G1-96	RHR Suction Valve Testing
GI-99	RCS/RHR Suction Valve Testing
61-101	BWR Water Level Redundancy
GI-103	Maximum Precipitation
G1-107	Main Transformer Failure
GI-110	Protection Devices on ESF Equipment
GI-115	Reliability of W Solid State Protection Systems
GI-116	Accident Management
GI-117	Allowable Outage Times
GI-118	Tendon Anchor Failure
GI-120	On-Line Testing of Protection Systems
GI-124	EFW Design
GI+127	Testing & Maintenance of Manual Valves
GI-132	RHR Pumps in Containment
67.5.1	Davis Besse Event SG Reassessment
67.5.2	Davis Besse Event SG Reassessment
122.1a	Davis Besse Event EFW Reliability
122.1b	Davis Besse Event EFW Reliability
122.1c	Davis Besse Event EFW Reliability
122.2	Initiate Feed & Blecd
125.1.3	Design Basis Event Long Term Actions
125.1.4	Design Basis Event Long Term Actions
125.1.5	Design Basis Event Long Term Actions
125.1.6	Design Basis Event Long Term Actions
125.11.7	Automatic FW Isolation
125.11.11	Design Basis Event Long Term Actions
125.11.13	Design Basis Event Long Term Actions
II.E.4.3	Containment Design Integrity Check
II.E.6.1	In-Situ Valve Testing