UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO THE SECOND TEN-YEAR INTERVAL INSERVICE INSPECTION PROGRAM PLAN

CONSUMERS POWER COMPANY

PALISADES NUCLEAR PLANT

DOCKET NO. 50-255

1.0 INTRODUCTION

Technical Specification 4.0.5 for the Palisades Nuclear Plant states that inservice inspection of American Society of Mechanical Engineers (ASME) Code Class 1, 2, and 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50.55a(g)(6)(i). Subsection 50.55a(a)(3) of 10 CFR states that alternatives to the requirements of paragraph (g) may be used if (i) the proposed alternatives would provide an acceptable level of quality or safety, or (ii) compliance with the specified requirements of this section would result in hardship or unusual difficulties without a compensating increase in the level of quality and safety.

Pursuant to 10 CFR 50.55a(g)(4), ASME Code Class 1, 2, and 3 components (including supports) shall meet the requirements, except the design and access provisions and the preservice examination requirements, set forth in the ASME Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," to the extent practical within the limitations of design, geometry, and materials of construction of the components. The regulations require that inservice examinations of components and system pressure tests conducted during the second ten-year interval comply with the requirements in the latest edition and addenda of Section XI of the ASME Code incorporated by reference in 10 CFR 50.55a(b) on the date twelve months prior to the start of the 120-month inspection interval, subject to the limitations and modifications listed therein. The components (including supports) may meet the requirements set forth in subsequent editions and addenda of the ASME Code incorporated by reference in 10 CFR 50.55a(b) subject to the limitations and modifications listed therein.

Pursuant to 10 CFR 50.55a(g)(5), if the licensee determines that conformance with an examination requirement of Section XI of the ASME Code is not practical for his facility, information shall be submitted to the Commission in support of that determination and a request made for relief from the ASME Code requirement. After evaluation of the determination, pursuant to 10 CFR 50.55a(g)(6)(i) or, alternatively, 10 CFR 50.55a(a)(3), the Commission may grant relief and may impose alternative requirements as it determines

is authorized by law, will not endanger life, property, or the common defense and security, and is otherwise in the public interest, giving due consideration to the burden upon the licensee that could result if the requirements were imposed on the facilty.

The licensee, Consumers Power Company, has prepared the Palisades Nuclear Plant Second Ten-Year Interval Inservice Inspection (ISI) Program and Plan to meet the requirements of the 1983 Edition, Summer 1983 Addenda of Section XI of the ASME Boiler and Pressure Vessel Code, except that the extent of examination of Class 2 piping welds has been determined by the 1974 Edition and Addenda through the Summer 1975 Addenda as required by 10 CFR 50.55a(b)(2)(iv)(B). The staff, with technical assistance from its Contractor, the Idaho National Engineering Laboratory (INEL), has evaluated the Second Ten-Year Interval Inservice Inspection Program, additional information related to the Program and Plan, and the requests for relief from certain ASME Code requirements determined to be impractical for Palisades Nuclear Plant during the second inspection interval.

2.0 EVALUATION

The ISI Program and Plan have been evaluated for (a) application of the correct Section XI Code edition and addenda, (b) compliance with examination and test requirements of Section XI, (c) acceptability of the examination sample, (d) compliance with prior ISI commitments made by the licensee, (e) correctness of the application of system or component examination exclusion criteria, and (f) adequate information in support of requests for relief from certain Section XI Code requirements deemed impractical by the licensee. The staff has determined that the licensee's ISI Program Plan reflects compliance with the requirements listed above.

The information provided by the licensee in support of requests for relief from certain Section XI requirements has been evaluated and the bases for granting relief from those requirements are documented in the attached INEL Technical Evaluation Report EGG-MS-9172. We concur with the findings and recommendations contained in the subject report. We have determined that relief is not required for Relief Request RR-1. Pursuant to 10 CFR 50.55a(g)(6)(i), relief is granted with conditions stated in the TER for Relief Requests RR-2 and RR-3. Pursuant to 10 CFR 50.55a(a)(3)(ii), relief is granted with conditions stated in the TER for Relief Request RR-4. Table 1 presents a summary of the reliefs requested and the status of the requests as determined by the staff.

3.0 CONCLUSION

The staff concludes that the Palisades Nuclear Plant Second Ten-Year Interval Inservice Inspection Program and Plan, with the additional information provided and the specific written relief, constitutes the basis for compliance with 10 CFR 50.55a(g) and Technical Specification 4.0.5 and is, therefore, acceptable.

TABLE 1

SUMMARY OF RELIEF REQUESTS

Relief Request Number	System or Component	Exam. Cat.	Item No.	Volume or Area to be Examined	Required Method	Licensee Proposed Alternative	Relief Request Status
RR-1	RPV Control Rod Drive	B-G-2	B7.80	Control rod drive (CRD) bolting other than the peripheral CRD housing bolting	VT-1 Visual	None. VT-1 visual exam when disassembled for maintenance	Relief not Required
RR-2	Class 1 Piping	B-J	B9.11 B9.12	Class 1 pressure retaining piping welds: PCS-42-RCL-1H-1 PCS-42-RCL-2H-1 PCS-30-RCL-1A, 16LU-1, 16LU-2, & 16 PCS-30-RCL-1B, 14LU-1, 14LU-2, & 14 PCS-30-RCL-2A, 15LU-1, 15LU-2, & 15 PCS-30-RCL-2B, 15LU-1, 15LU-2, & 15	Volumetric and surface exam	Automated 100% volumetric exam from piping ID	Granted with conditions stated in Technical Evaluation Report
RR-3	Reactor Coolant Pumps	B-L-1	B12.10	RC pump casing welds	Volumetric Exam	None	Granted with conditions stated in Technical Evaluation Report

TABLE 1

SUMMARY OF RELIEF REQUESTS

Relief Request Number	System or Component	Exam. Cat.	Item No.	Volume or Area to be Examined	Required Method	Licensee Proposed Alternative	Relief Request Status
RR-4	Reactor Coolant Pumps	B-L-2	B12.20	RC pump casing internal surfaces	VT-3 visual exam	None. VT-3 visual exam to maximum extent practical if pump disassembled for maintenance	Granted with condition stated in Technical Evaluation Report