

Docket No. 50-219
LS05-83-12-006

December 7, 1983

Mr. P. B. Fiedler
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Fiedler:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - TMI ACTION PLAN ITEM
II.B.2, DESIGN REVIEW OF PLANT SHIELDING

Oyster Creek Nuclear Generating Station

The staff has completed a preliminary review of the corrective actions taken for access to vital areas at Oyster Creek Nuclear Generating Station, relative to NUREG-0737, Item II.B.2. Details of this review are documented in NRC Region I Inspection Report 50-219/83-13 and were based, in part, upon licensee letters to NRC dated January 4, 1980, and April 10, 1980.

To complete our evaluation, we will require the additional information as stated in the enclosure. Therefore, we request that you provide this information within 30 days of receipt of this letter.

The reporting and/or recordkeeping requirements contained in this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

Sincerely,

ORIGINAL SIGNED BY
Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosure:

RAI
cc: w/enclosure See next page

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Mr. P. B. Fiedler

cc

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Licensing Supervisor
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Enclosure

Request For Additional Information

NUREG-0737, Item II.B.2, Design Review of Plant Shielding

for Oyster Creek Nuclear Generating Station

(NRC Region I Inspection Report 50-219/83-13)

1. Source Terms were identified as being derived from GE data and documented in Computer Run SNUMB-7007S dated November 9, 1979. Who performed this calculation, and what type of code was used (for example, ORIGEN)?
2. Airborne concentrations were presented in Enclosure 3 to the January 4, 1980 letter to NRC in units of mc/cc. What are the correct units, and what method was used to derive these values?
3. For the appropriate airborne concentrations decided to potentially exist in the Reactor Building, show the expected contribution of this source (specifically the upper, unshielded portion of the Reactor Building) to time-dependent dose rates and integrated post-accident dose rates in vital areas (including control room) identified by question 4 below.
4. Identify those areas considered as vital (either continuous or intermittent occupancy) at Oyster Creek, including as a minimum: control room, TSC, Sampling and Analysis Centers, Security Building, Diesel Generator Building and SGTS Filters. For vital areas of intermittent occupancy, specify when and how long access would be required. For all vital areas, provide expected dose rates as a function of time post-accident (and the appropriate contributing sources) and the integrated whole body dose expected over the course of an accident for comparison with the 5 Rem guideline.
5. Explain the reasoning contained in the April 15, 1983 GPU letter to NRC (P. Fielder to D. Eisenhut) entitled "Cycle 10 Refueling Outage Workload", for cancellation of the commitment for a SGTS Filter Tie-In. Specifically, what analyses have been performed to conclude that: (1) a single SGTS filter train is capable of handling (without changeout) effluent loading associated with an excessive MSIV leakage accident, and (2) whether radiological source contribution need be considered for any vital areas (such as the Security Building) from such a source?