

## (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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REPORT SOURCE

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DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | As stated in the original LER, unit 2 was in mode 1 at 98% Rx power when it was

0 3 | discovered at 0815 CST on 02/25/83 that the containment pressure was greater than

0 4 | the +.3 psi relative to the annulus (approximately .4 psi relative to the annulus).

0 5 | The original event required entry into LCO 3.6.1.4. There was no effect on public

0 6 | health or safety. Previous occurrences - none.

SYSTEM CODE S A		CAUSE CODE E		CAUSE SUBCODE F		COMPONENT CODE V A L V O P		COMP. SUBCODE D		VALVE SUBCODE Z	
LER/RQ REPORT NUMBER 17		EVENT YEAR 8 3		SEQUENTIAL REPORT NO. 0 2 7		OCCURRENCE CODE 0 3		REPORT TYPE L		REVISION NO. 1	
ACTION TAKEN X	FUTURE ACTION C	EFFECT ON PLANT Z	SHUTDOWN METHOD Z	HOURS 0 0 0 0	ATTACHMENT SUBMITTED Y	NPG-4 FORM SUB. N	PRIME COMP. SUPPLIER L	COMPONENT MANUFACTURER X 9 9 9			

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | At 0917 on 02/25/83, the containment pressure was returned below the +.3 psi limit  
1 1 | by venting containment. A leak in a train 'B' essential air line was suspected and  
1 2 | the line was isolated until a unit shutdown allowed investigation and repair. An  
1 3 | inspection on 07/20/83 discovered the air regulator to pressurizer power-operated  
1 4 | relief valve 2-PCV-68-334, to have a ruptured diaphragm. The air regulator was  
isolated.

8311010329 831021  
PDR ADCK 05000328  
S PDR

Name of Preparer: G. B. Kirk /M. R. Harding

Phone: (615) 870-6422

Sequoyah Nuclear Plant

LER SUPPLEMENTAL INFORMATION

SQRO-50-328/83027, Revision 1      Technical Specification Involved: 3.6.1.4

Reported Under Technical Specification: 6.9.1.13.b

Date of Occurrence: 02/25/83      Time of Occurrence: 0815 CST

Identification and Description of Occurrence:

At 0815 CST on 02/25/83, it was discovered that the containment pressure was greater than the +.3 psi relative to the annulus (approximately .4 psi relative to the annulus).

Conditions Prior to Occurrence:

Unit 2 in mode 1 at 98% reactor power.

Apparent Cause of Occurrence:

Investigation on 07/20/83 discovered the air regulator to pressurizer power-operated relief valve had a ruptured diaphragm.

Analysis of Occurrence:

The original event required entry into LCO 3.6.1.4.

Corrective Action:

At 0917 on 02/25/83, the containment pressure was returned below the +.3 psi limit by venting containment. A leak in a train 'B' essential air line was suspected and the line was isolated, requiring entry into LCO 3.4.3.2, until a unit shutdown would allow investigation and repair. Following a unit trip on 07/18/83, an inspection on 07/20/83 discovered the air regulator to pressurizer power-operated relief valve, 2-PCV-68-334, had a ruptured diaphragm. The air regulator was isolated. Since the PORVs are being replaced with solenoid operated type valves during the present unit 2 refueling outage, the air regulator will be removed.

Failure Data:

None.

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

October 21, 1983

83 OCT 27 410:48

Mr. James P. O'Reilly, Director  
U.S. Nuclear Regulatory Commission  
Suite 2900  
101 Marietta Street, NW.  
Atlanta, Georgia 30303

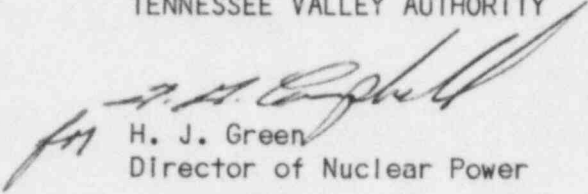
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 2 - DOCKET  
NO. 50-328 - FACILITY OPERATING LICENSE DPR-79 - REPORTABLE OCCURRENCE  
REPORT SQRO-50-328/83027 - REVISION 1

The enclosed report is a supplement to my letter to you dated March 25,  
1983 concerning the exceeding of the containment internal pressure limit.  
This report is submitted in accordance with Sequoyah unit 2 Technical  
Specification 6.9.1.13.b.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

  
H. J. Green  
Director of Nuclear Power

Enclosure

cc (Enclosure):

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Washington, D.C. 20555

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Atlanta, Georgia 30339

NRC Inspector, Sequoyah

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