NRC FORM 366 (7-77)

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LICENSEE EVENT REPORT

U.S. NUCLEAR REGULATORY COMMISSION

CONTROL BLOCK
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CON'T BEPORT LIG 0 5 0 - 0 3 0 2 0 9 2 2 8 3 8 1 0 2 0 8 3 9 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (1) O [2] [On September 22, 1983, while reviewing chemistry analysis, it was dis-
covered that no documentation was available to verify the performance
of Surveillance Requirement 4.6.2.2.b.2 for July, 1982. This surveil-
I lance was satisfactorily performed in January, 1983. This is the first
time that surveillance 4.6.2.2.b.2 was missed.
7 8 9 SYSTEM CAUSE CAUSE COMPONENT CODE COMP. VALVE 80 CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE
$\begin{bmatrix} S \\ H \\$
TO REPORT L RISI LI LOLAL OL LOLAL OL LOLAL OL
NUMBER 21 22 23 24 26 27 28 29 30 31 32
TAKEN ACTION ONPLANT METHOD HOURS (2) ATTACHMENT NPRD-4 PRIME COMP. COMPONENT Z 18 X 19 Z 20 Z 21 0 0 0 0 V 23 N 24 Z 25 Z 9 9 9 26
33 34 35 36 37 40 41 42 43 44 47 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
10 This surveillance requirement was evidently overlooked during the period
that chemistry control procedures were being revised. Chemistry records
[1] will be reviewed to identify any other omissions that occurred due to
13 L the procedure revisions occurring during Mid-1982. This review will be
completed by January 1, 1984.
7 3 9 FACILITY STATUS * POWER OTHER STATUS 30 METHOD OF DISCOVERY DISCOVERY DESCRIPTION 32 DISCOVERY DESCRIPTION 32 DISCOVERY DESCRIPTION 32
7 8 9 10 12 13 A B 31 Surveillance Procedure Review
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) 1 6 9 10 11 N/A LOCATION OF RELEASE (36)
PERSONNEL EXPOSURES NUMBER
1 7 8 9 PERSONNEL INJURIES 13 80
7 8 0 11 12 LOSS OF OR DAMAGE TO FACILITY 43 TYPE DESCRIPTION 80
19 JZ 42 N/A
PUBLICITY 80 ISSUED DESCRIPTION 45 NRC USE ONLY
Image: Normal state N/A Image: Normal state Imag
NAME OF PREPARER P.G. Hughes PHONE (904) 795-3802
B311010214 B31020 PDR ADOCK 05000302 S PDR ZE72

SUPPLEMENTARY INFORMATION

REPORT NO.:50-302/83-040/03L-0FACILITY:Crystal River Unit 3REPORT DATE:October 20, 1983OCCURRENCE DATE:September 22, 1983

IDENTIFICATION OF OCCURRENCE:

The semi-annual surveillance of the sodium hydroxide concentration (NaOH) in the Building Spray Additive System was not performed as required by Technical Specification 4.6.2.2.b.2.

CONDITIONS PRIOR TO OCCURRENCE:

Mode 1 (73% full power)

DESCRIPTION OF OCCURRENCE:

On September 22, 1983, Florida Power Corporation was reviewing the sodium hydroxide concentration data as a follow-up to LER 83-024. (LER 83-024 identified that FPC has erroneously been equating 1.0 mg per liter with 1.0 ppm.) Responsible personnel were unable to provide objective evidence that the semi-annual concentration determination had been performed as required during July, 1982.

DESIGNATION OF APPARENT CAUSE:

During July, 1982, chemistry control procedures were being revised en masse. Due to the complexity of these revisions, the requirement to perform the NaOH analysis was evidently overlooked.

ANALYSIS OF OCCURRENCE:

The next semi-annual concentration analysis, performed January, 1983, indicated that the concentration was within Technical Specification limits (as they were understood at that time).

CORRECTIVE ACTIONS:

Other chemistry analyses that were required to be performed during July and August, 1982, will be reviewed to identify any additional omissions by January 1, 1984.

FAILURE DATA:

This was the first time that surveillance 4.6.2.2.b.2 was missed and the third report under Specification 3.6.2.2.

33 OCT 27 AID: 46



October 20, 1983 3F1083-17

Mr. James P. O'Reilly Regional Administrator, Region II Office of Inspection & Enforcement U.S. Nuclear Regulatory Commission 101 Marietta Street N.W., Suite 2900 Atlanta, GA 30303

Subject: Crystal River Unit 3 Docket No. 50-302 Operating License No. DPR-72 Licensee Event Report No. 83-040

Dear Mr. O'Reilly:

Enclosed is Licensee Event Report No. 83-040 and the attached supplementary information sheet, which are submitted in accordance with Technical Specification 6.9.1.9(b).

Should there be any questions, please contact this office.

Sincerely, Vestaler

G.R. Westafer Manager Nuclear Operations Licensing and Fuel Management

AEF:jcf

Enclosure

cc: Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555

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