



Consumers
Power
Company

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October 25, 1983

Dennis M Crutchfield, Chief
Operating Reactors Branch No 5
Nuclear Reactor Regulation
US Nuclear Regulatory Commission
Washington, DC 20555

DOCKET 50-155 - LICENSE DPR-6 -
BIG ROCK POINT PLANT - AN UPDATED PROPOSED
TECHNICAL SPECIFICATION CHANGE REQUEST -
SPENT FUEL RACK MODIFICATION

On June 26, 1979, Consumers Power Company submitted to the NRC staff an application including proposed changes to the Technical Specifications, for expansion of the spent fuel pool storage capacity at Big Rock Point. The Description and Safety Analysis and Environmental Impact Evaluation in support of this application was sent to the NRC staff on April 23, 1979. The NRC staff determined that the June 26 application involved a significant hazards consideration and a Notice of Opportunity for Hearing was published in the Federal Register on July 17, 1979.

On May 15, 1981, the NRC issued a Safety Evaluation Report (SER) and Environmental Impact Appraisal in support of the proposal to amend the Big Rock Point License to permit expansion of the spent fuel pool storage capacity. A Supplement to the Safety Evaluation Report was also issued by the NRC on September 30, 1983. Both the original May 15, 1981 SER and the September 30, 1983 Supplement specified additional Technical Specification requirements. The purpose of this letter is to formalize these additional requirements by updating our original June 26, 1979, Technical Specification Change Request.

The changes specified by the June 26, 1979 request are incorporated in the attachment. Additional requirements specified by the NRC Safety Evaluation Report, dated May 15, 1981 and the Supplement to the Safety Evaluation Report dated September 30, 1983 are also included. These additional requirements address the following:

- 1) A limit of 28.3 grams of uranium-235, or equivalent, per axial centimeter of fuel assembly

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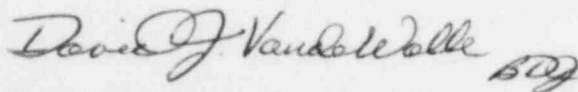
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- 2) Availability of make-up water to the spent fuel pool in the event containment cannot be entered.
- 3) A limit of 150°F maximum pool water temperature prior to restarting the reactor following any addition of heat load to the spent fuel pool.
- 4) A requirement for functional testing of the trip mechanism of the spent fuel transfer cask redundant support assembly. (The May 15, 1981 SER identifies that the change was originally submitted by letter dated August 14, 1980. It is incorporated in the attachment as originally submitted.)

This proposed change was classified as Class IV pursuant to 10CFR170.22. A check for \$12,300 was attached to our June 26, 1979 request.



David J Vandewalle
Director of Nuclear Licensing

CC Administrator, Region III, USNRC
NRC Resident Inspector-Big Rock Point

Attachment