

LICENSEE EVENT REPORT

CONTROL BLOCK:

| | | | | | | |
|--|--|--|--|--|--|-----|
| | | | | | | (1) |
|--|--|--|--|--|--|-----|

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

| | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
|---|---|---------------|---|---|---|---|---|----|----|----------------|---|---|---|---|---|---|---|---|---|----|----|--------------|---|---|---|---|----|----|--------|--|
| 0 | 1 | N | Y | N | M | P | 1 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | | 5 | | |
| 7 | 8 | LICENSEE CODE | | | | | | 14 | 15 | LICENSE NUMBER | | | | | | | | | | 25 | 26 | LICENSE TYPE | | | | | 30 | 57 | CAT 58 | |

CON'T

REPORT SOURCE 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

0 1 L 6 0 5 0 0 0 2 2 0 7 0 8 2 6 8 3 8 0 9 2 6 8 3 9

DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During power operation, while performing a channel test, one of the two inadequate
0 3 | core cooling reactor core level indicators failed the surveillance test. Consequences
0 4 | were minimal since the system only activates during a severe accident, and since
0 5 | the redundant component was proven operable, and all other level detectors were
0 6 | also operable. The channel was out of service for 3 days.
0 7 |
0 8 |

| | | | | | | | | | | | | | | | |
|----------------------|----|---------------------------|----|-----------------------------------|----|-------------------------------|----|------------------|----|---------------------------|----|--------------------|----|--------------------|----|
| SYSTEM CODE 0 9 | | CAUSE CODE I E | | CAUSE SUBCODE E | | COMPONENT CODE I N S T R U | | | | | | COMP. SUBCODE I | | VALVE SUBCODE Z | |
| 7 | 8 | 9 | 10 | 11 | 12 | 13 | 14 | 15 | 16 | 17 | 18 | 19 | 20 | 21 | 22 |
| LER/RO REPORT NUMBER | | EVENT YEAR 8 3 | | SEQUENTIAL REPORT NO. 0 2 4 | | OCCURRENCE CODE 0 3 | | REPORT TYPE L | | REVISION NO. 0 | | | | | |
| 23 | 24 | 25 | 26 | 27 | 28 | 29 | 30 | 31 | 32 | 33 | 34 | | | | |
| ACTION TAKEN G | | FUTURE ACTION Z | | EFFECT ON PLANT Z | | SHUTDOWN METHOD Z | | HOURS 0 0 0 | | ATTACHMENT SUBMITTED N | | | | | |
| 35 | 36 | 37 | 38 | 39 | 40 | 41 | 42 | 43 | 44 | 45 | 46 | | | | |
| NPRD-4 FORM SUB. | | PRIME COMP. SUPPLIER N | | COMPONENT MANUFACTURER G 0 8 0 | | | | | | | | | | | |
| 47 | 48 | 49 | 50 | 51 | 52 | | | | | | | | | | |

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The computer based system displayed incorrect data as a result of an extremely high

1 1 value in one register. The cause of this high value is believed to have been bad

1 2 contacts between the multiplier board and the data logger chassis. The register was

1 3 zeroed, the system was re-initialized, and the channel test was successfully completed.

1 4 The surveillance interval has been increased pending resolution of the problem.

| | | | | | | | | | | | | | | | | | |
|---|---|---|-------------------------------|-------------------|----|--------------------|----|----|---------------------|----|--------------|---------------------|--|--|-----------------------|--|--|
| 7 | 8 | 9 | FACILITY STATUS | | | % POWER | | | OTHER STATUS | | | METHOD OF DISCOVERY | | | DISCOVERY DESCRIPTION | | |
| 1 | 5 | E | 28 | 1 | 0 | 0 | 29 | NA | B | 31 | Surveillance | | | | | | |
| 7 | 8 | 9 | ACTIVITY CONTENT | | | AMOUNT OF ACTIVITY | | | LOCATION OF RELEASE | | | | | | | | |
| 1 | 6 | Z | 33 | Z | 34 | NA | 35 | NA | | | | | | | | | |
| 7 | 8 | 9 | PERSONNEL EXPOSURES | | | DESCRIPTION | | | | | | | | | | | |
| 1 | 7 | 0 | 0 | 0 | 37 | Z | 38 | NA | | | | | | | | | |
| 7 | 8 | 9 | PERSONNEL INJURIES | | | DESCRIPTION | | | | | | | | | | | |
| 1 | 8 | 0 | 0 | 0 | 40 | NA | | | | | | | | | | | |
| 7 | 8 | 9 | LOSS OF OR DAMAGE TO FACILITY | | | DESCRIPTION | | | | | | | | | | | |
| 1 | 9 | Z | 42 | NA | | | | | | | | | | | | | |
| 7 | 8 | 9 | PUBLICITY | | | DESCRIPTION | | | | | | | | | | | |
| 2 | 0 | N | 44 | 8310310359 830926 | | | | | | | | | | | | | |
| 7 | 8 | 9 | ISSUED | | | PDR | | | | | | | | | | | |
| 2 | 0 | | | | S | | | | | | | | | | | | |
| 7 | 8 | 9 | | | | | | | | | | NRC USE ONLY | | | | | |
| 7 | 8 | 9 | | | | | | | | | | | | | | | |

NAME OF PREPARER

D.T. Kesel

PHONE:

(315) 349-2611

September 26, 1983

Dr. Thomas E. Murley
Regional Administrator
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

RE: Docket No. 50-220
LER 83-24

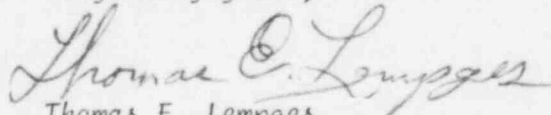
Dear Dr. Murley,

In accordance with Nine Mile Point Nuclear Station Unit #1 Technical Specifications, we hereby submit the following Licensee Event Report:

83-24 which is being submitted in accordance with
Section 6.9.2(b)2, Conditions leading to
operation in a degraded mode permitted by a
limiting condition for operation or plant
shutdown required by a limiting condition
for operation.

This report was completed in the format designated in NUREG-0161,
dated July 1977.

Very truly yours,



Thomas E. Lempges
Vice President
Nuclear Generation

TEL/jm

Attachments (3 copies)

cc: Director, Office of I&E (30 copies)
Director, Office of MIPC (3 copies)

IE22
11