

50-302



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 14, 1995

Mr. Percy M. Beard, Jr.
Senior Vice President,
Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear
Licensing
15760 W Power Line Street
Crystal River, Florida 34428-6708

Dear Mr. Beard:

SUBJECT: CRYSTAL RIVER NUCLEAR GENERATING PLANT UNIT 3 - EMERGENCY DIESEL
GENERATOR MAINTENANCE AND TESTING (TAC NO. M86966)

We have reviewed your letter dated September 9, 1994, which provided a summary of the post-maintenance testing results on your facility emergency diesel generators (EDG). We find your results acceptable as summarized below.

In 1990, you upgraded the EDGs to increase their capacity from 2700 to 2850 kw. Following the upgrade, you performed load testing on the EDGs. The test included voltage measurements at selected locations at 480 and 120 volts. The test results did not meet all of the acceptance limits of Regulatory Guide 1.9. By letter dated September 10, 1993, you submitted to us the test results to demonstrate that the voltage dips were not significant and concluded that the motors, control devices, and protective devices powered by the EDG would function as designed.

During the recent ninth refueling outage, you performed post-maintenance testing on both EDGs. The voltage response tests conducted in conjunction with the Combined Safety Injection Actuation Signal and Loss of Offsite Power Test resulted in improved voltage values from the previous tests. For the modified test, you actuated the engineered safety features and simulated an undervoltage on the engineered safeguards bus simultaneously instead of successively.

By letter dated September 9, 1994, you provided a summary of the test results. The test results showed improved diesel performance which you attributed to the modified test being more representative of the grid response following a large break loss of coolant accident. On the basis of the test results, you concluded that the equipment powered by the diesels would perform their intended function under emergency conditions.

Your results also showed, in certain cases, some short-duration voltage peaks i.e., over voltage conditions. You do not believe that this would result in any adverse consequences, on the basis that any immediate failure due to flash-over or insulation breakdown as a result of voltage peaks would be detected by the failure of the associated component. A shortened component life due to increased voltage stress is unlikely due to the short duration and infrequent testing (only once every 24 months).

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We find your responses acceptable. Further staff review, if any, will be performed by future inspection or audits. This completes our effort under this Technical Assignment Control (TAC) number and, therefore, TAC No. M86966 is closed.

Sincerely,



L. Raghavan, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

cc: See next page

Docket No. 50-302

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Crystal River Unit No. 3
Generating Plant

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