"PCP" Date February 28, 1984

CALLAWAY PLANT

PROCESS CONTROL PROGRAM

PCP

RESP. DEPT.	Redweste	PREPARED BY	G.W. Harr	ilton
APPROVED BY	- Titapa	stud_	DATE 3/2	84
DATE ISSUED	3-5-84	- 0		

This procedure contains the following:

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Attachments	1	through	5
Appendices	and and an and a	through	
Checklist		through	

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"PCP" Date February 28, 1984

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CALLAWAY PLANT

PROCESS CONTROL PROGRAM MANUAL

(PCP)

1.0	PURPOSE	AND	SCOPE

- 1.1 The purpose of the Process Control Program (PCP) for radwaste solidification is to provide reasonable assurance and documentation of the complete solidification of processed radioactive wastes and of the absence of free standing water of the processed waste within the limits as set forth in 10CFR20, 10CFR61, 10CFR71, the Radiological Effluent Technical Specifica-1 tions, Federal and State regulations, burial ground requirements, and other requirements 1 governing the disposal of the radioactive 1 wastes.
- 1.2 The PCP contains the sampling, tests, analyses, and formulation determination by which waste classification and solidification of radioactive wastes from liquid systems is assured.
 - 2.0 DEFINITIONS
 - 2.1 <u>Solidification</u> The conversion of wet wastes into a form that meets shipping and burial ground requirements.
- 2.2 <u>Batch</u> A specified quantity of waste material requiring soldification any portion of which would have the same physical and chemical characteristics as the whole.
- ! 2.3 Waste Classification

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The determination of waste class as outlined in 10CFR61 by radionuclide isotopic analysis and/or correlation with measured nuclides.

! 3.0 10CFR61 COMPLIANCE METHODOLOGY

- ! 3.1 All solidified radioactive wastes generated ! at the Callaway Plant shall meet the ! requirements of Title 10 Code of Federal ! Regulations Part 61 (10CFR61).
- ! 3.2 Waste Classification (10CFR61.55) requirements shall be met by the methodology established in RDP-ZZ-00004, Waste ! Classification.
- ! 3.3 Waste Characterization (10CFR61.56) requirements shall be met by a combination of the methodology stated in the Process Control Frogram (Section 6.0) and related Callaway Plant Operating Procedures. Stability requirements stated in 10CFR61.56 (b) and by burial site shall be met and ensured prior to shipping solidified radioactive waste offsite for burial.

4.0 PRECAUTIONS AND LIMITATIONS

- 4.1 All samples shall be handled in accordance with applicable Callaway Plant procedures and in keeping with ALARA principles.
- 4.2 Test samples containing radioactive waste shall be disposed of as radioactive waste.
- 5.0 COLLECTION AND ANALYSIS OF SAMPLES
- 5.1 General Requirements
- 5.1.1 As required by Radiological Effluent Technical Specification 3.11.3, the PCP shall be used to verify the solidification of at least one representative test sample from at least every tenth batch of each type of wet radioactive waste processed.

- 5.1.2 For the purpose of the Callaway Plant PCP, a batch shall consist of a particular amount of liquid wastes/sludges requiring solidification (ie., the amount of waste content within a tank requiring solidification, or, the amount of waste content within two tanks requiring solidification if the contents of the two tanks are to be solidified together within a common drum). If new material is added to a tank's contents which is currently being processed, a new batch is created and further sampling must be performed prior to solidification.
- 5.1.3 If any sample fails to solidify, solidification of the batch under test shall be suspended until such time as additional test samples can be obtained, alternative solidification parameters can be determined in accordance with the Process Control Program, and a subsequent test verifies solidification. Solidification of the batch may then be resumed using the alternative solidification parameters determined.
- 5.1.4 If the initial test sample from a batch of waste fails to verify solidification, then representative test samples shall be collected from each consecutive batch of the same type of waste until three (3) consecutive test specimens demonstrate solidification using the alternate parameters of step 6.3.3.
- 5.1.5 For high activity wastes, where handling samples could result in personnel radiation exposures which are inconsistent with ALARA principles, representative non-radioactive samples may be test solidified. These samples shall be as close to the actual waste and chemical properties as possible. Typical unexpended mixed bead resin may be used to simulate the spent bead resin.

- ! 5.1.6 Where practical, all chemicals used to condition or solidify waste (or simulated waste) in solidification tests shall be identical to the actual chemicals to be used in full scale solidification.
 - 5.2 Collection of Samples
 - 5.2.1 WASTE SOLIDIFICATION DATA SHEET (Attachment 1)
 - 5.2.1.1 The WASTE SOLIDIFICATION DATA SHEET will contain pertinent information on the characteristics of the test sample solidified in order to verify solidification of subsequent batches of similar waste without retesting.
 - 5.2.1.2 The test sample data for waste shall include, but is not limited to: the type of wastes solidified; percent total dissolved solids; pH; volume of sample; amount of oil in the sample; the waste to portland cement ratio; the portland cement to sodium metasilicate ratio.
- 5.2.1.3 The WASTE SOLIDIFICATION DATA SHEET will include the batch number, batch volume, waste type, waste classification, total waste received, total portland cement added, total sodium metasilicate added and the date solidified.
 - 5.2.1.4 If waste pretreatment is necessary prior to actual batch solidification per the results of step 6.1, the agent used and amount added shall be denoted in the remarks section of the WASTE SOLIDIFICATION DATA SHEET.
 - 5.2.2 Taking Samples
- 5.2.2.1 A sample(s) of the waste tank's contents requiring solidification must be taken in order to determine the actual process formulation for solidification, any pretreatment of the waste needed prior to solidification, and the waste classification of the tank contents to be solidified.

- 5.2.2.1.1 Sample sizes, as determined by the Radwaste Department, snall be compatible with the standard size samples used for radioactivity and chemical analysis.
- 5.2.2.1.2 If the radioactivity levels are too high to permit full size samples to be taken, then smaller samples shall be taken with the results corrected accordingly.
- 5.2.2.2 Sufficient sampling lead time should be allotted prior to the planned waste solidification of a batch to allow adequate time to complete the required testing and verification of solidification, as applicable.
- 5.2.2.3 The contents of the waste tanks that are to be solidified should be recirculated (mixed) prior to sampling to ensure that a representative sample is obtained.
- 5.2.2.4 If the contents of more than one tank are to be solidified in the same drum, then representative samples of each tank should be drawn.
- 5.2.2.4.1 These samples should be of a sufficient composition that if "X" percent of the total waste to be solidified is to be taken from one of the tanks, then the sample taken from that tank should be the same percentage in the composite sample.
- 5.2.2.4.2 The samples taken of each should be mixed in the proper proportions to yield a standard size sample as described in 5.2.2.1.1.
- 5.3 Chemical Analysis of Waste Samples
- 5.3.1 Evaporator bottoms and chemical wastes shall be analyzed for total dissolved solids, oil content, boric acid concentration and pH and the results recorded on the WASTE SOLIDIFI-CATION DATA SHEET.

5.3.2	Spent resin beads and charcoal shall be characterized by analyzing the water sur- rounding the beads and charcoal for oil con- tent and pH. Boric acid concentration of spent resin beads shall be determined by an assay of the boric acid separated from the anion bead resin. The results shall be recorded on the WASTE SOL DIFICATION DATA SHEET.
	SHEET.

- 5.4 Radiochemical Analysis of Waste Samples
- 5.4.1 A gamma isotopic analysis will be performed on each batch to be solidified.
- 5.4.2 This analysis will be used to determine acceptability for solidification as well as waste classification of the waste to be solidified (see Step 7.0).
 - 5.4.3 The results of the gamma isotopic analysis shall be attached to the WASTE SOLIDIFICA-TION DATA SHEET.
 - 6.0 TEST SOLIDIFICATION AND ACCEPTANCE CRITERIA
 - 6.1 Waste Conditioning
 - 6.1.1 Prior to the test sample solidification, the pH of the sample shall be adjusted to a range of 7 to 10.
 - 6.1.1.1 Should adjustment be necessary, the agent and quantity used shall be recorded on the WASTE GOLIDIFICATION DATA SHEET.
- 6.1.2 If oil is present in quantities greater than 1% by volume, dilution of the batch to below 1% is required prior to solidification.
 - 6.1.2.1 If reduction of the oil content is impossible or impractical, solidification of the batch shall not be attempted using the Stock Solidification System, but shall be accomplished using a bulk processing method.

6.2 Test Solidification

1. 1

- 6.2.1 Whenever pretreatment of a batch is necessary, the waste sample shall have the required pretreatment accomplished prior to the test solidification.
- 6.2.2 Prepare the test solidification vessel (suitably sized disposable beaker) with a mixing device.
- 6.2.3 Transfer a known representative volume of the waste to the test soldification container.
- 6.2.4 Add the appropriate proportional amount of portland cement and sodium metasilicate, as applicable, determined from the appropriate attachments (3, 4 and/or 5) found at the back of this manual.
- 6.2.5 Initiate mixing of the waste, portland cement and sodium metasilicate. After 10 (10) minutes of mixing or when a homogenous mixture is obtained, allow the waste to stand for a minimum of 30 minutes.
- 6.2.6 If any free liquid is observed on the top of the sample, decant the liquid into a clear volumetric beaker and record the amount of liquid transferred. Calculate the percent of free liquid and record the data on the WASTE SOLIDIFICATION DATA SHEET.
- ! 6.3 Test Solidification Acceptability

6.3.1 The test sample solidification will be considered acceptable from a free liquid standpoint if the amount of free liquid is the lessor of either 1% by volume or burial site limits.

-7-

- 6.3.2 The test sample solidification will be considered acceptable from a solid mass standpoint (i.e., structural stability) if it is evident from it's physical appearance that the solidified waste maintains its shape when removed from the container.
- 6.3.3 If either or both of the above checks fail to meet the stated criteria, alternative solidification parameters must be determined before solidification can proceed.
- 6.3.4 If the initial test solidification of a batch is unacceptable, then a representative sample shall be test solidified on each subsequent batch of the same type of waste until three consecutive test samples verify solidification using the alternate parameters of step 6.3.3.
- 6.3.5 If a test sample fails to provide acceptable soldification of waste, mix equal volumes of dry cement and water to ensure that the problem is not due to a bad batch of cement.
- 7.0 PROCESS FORMULATIONS

Prior to actual solidification of the waste to be solidified, classification and acceptability for near-surface disposal shall be determined as per RDP-ZZ-00004, WASTE CLASSIFICATION. This will involve calculating the activity within the projected waste drums using the isotopic analysis determined in step 5.4 and the solidification formulas determined below.

7.1 Spent Resins

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7.1.1 Solidify spent resins per formulas stated in Attachment 3, Spent Resin Solidification, and operating sequences specified in RTN-HC-00005, Decanting and Drumming Station Operation. 7.1.2 If the chemical analysis from Step 5.3 yields boric acid concentration in excess of <u>3%</u> in the spent resin to be solidified, determine the amount of sodium metasilicate required for solidification from Attachment 4, Borated Waste Solidification, and add this to the formula(s) in Step 7.1.1.

7.2 Chemical Drain Tank Wastes

- 7.2.1 If the chemical analysis from step 5.3 yields boric acid concentration in excess of <u>3%</u> in the chemical waste to be solidified, solidify chemical wastes in accordance with the formulation determined in Attachment 4, Borated Waste Solidification, and operating sequences specified in RTN-HC-00005, Decanting and Drumming Station Operation, and RTN-HR-00008, Chemical Drain Tank Operation.
- 7.2.2 If the chemical analysis from step 5.3 yields less than <u>3%</u> boric acid concentration, solidify chemical wastes in accordance with the formulation determined using Attachment 5, Non-Borated Waste Solidification, and operating sequences specified in RTN-HC-00005, Decanting and Drumming Station Operation, and RTN-HB-00008, Chemical Drain Tank Operation.

7.3 Evaporator Bottoms

7.3.1 If the chemical analysis from step 5.3 yields boric acid concentration in excess of 3% in the evaporator bottoms to be solidified, solidify evaporator bottoms in accordance with the formulation determined using Attachment 4, Borated Waste Solidification, and operating sequences specified in RTN-HC-00005, Decanting and Drumming Station Operation, and RTN-HC-00012, Evaporator Bottoms Tank Operation.

- 7.3.2 If the chemical analysis from step 5.3 yields less than <u>3%</u> boric acid concentration, solidify evaporator bottoms in accordance with the formulation determined using Attachment 5, Non-Borated Waste Solidification, and operating sequences specified in RTN-HC-00005, Decanting and Drumming Station Operation, and RTN-HC-00012, Evaporator Bottoms Tank Operation.
- 8.0 SOLIDIFYING AGENTS

As part of the operational Process Control Program, the following requirements shall be set forth in the purchase specifications for the cement and sodium metasilicate.

- 8.1 Cement
- 8.1.1 Portland cement ASTM C-150, Type II or Type III
- 8.1.2 The tests listed in Attachment 2 may be used to differentiate between types I, II and III of Portland cement.
- 8.2 Sodium Metasilicate
- 8.2.1 Sodium Metasilicate, anhydrous, granular (crystalline), commercial grade.
- 9.0 REFERENCES
- 9.1 ASTM Standards Part 13
- 9.2 STOCK EQUIPMENT COMPANY, General Process Control Program
- 9.3 RTN-HC-00005
- 9.4 RTN-HC-00012
- 9.5 RTN-HB-00008
- 9.6 10CFR20

9.7 10CFR61

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5.0

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- 9.8 10CFR71
- 9.9 Reg. Guide 8.8
- 9.10 RDP-ZZ-00004
- 9.11 Radiological Effluent Technical Specification 3.11.3

WASTE SOLIDIFICATION DATA SHEET

Batch No. : Sample No. :	
Date: Prepared By:	

I. Prior Batch Sample Solidification

Verify prior batch sample solidification performed. Check one below and show date and batch number of sample.

	Evaporator Bottoms (Prin	ary Date:	Batch No.
_	Evaporator Bottoms (Seco	ondary) Date:	Batch No.
	Chemical Wastes	Date:	Batch No.
-	Resins (Primary)	Date:	Batch No.
	Resins (Secondary)	Date:	Batch No.
	Other	Date:	Batch No

II. Batch Sample Analysis

A. Lab Results

III.

	Sample Volume			
	pH of Waste	Is less than 7 or greater than 10 - refer to Section 6.1.1 of PCP.		
	Percent Total Dissolved Solids			
	Boric Acid Concentration			
	Specific Gravity			
	Waste Oil Content	If greater than 1% refer to Section		
	Radionuclide Content	_ Attach copy of isotopic gamma analysis		
	Waste Classification	printout.		
в.	Waste Pretreatment			
	pH - Identify agent used and quantity t	to adjust pH of the waste in the sample.		
	Agent/Quantity	Adjusted pH value		
		Date:		
Sam	ple Solidification (Required on at leas radioactive waste.)			
Α.	Test Solidification Formula			
	Volume of Waste:			
	Volume of Cement:			

Volume of Sodium Metasilicate:

Waste/Cement Ratio:

	Cement/Sodium Metasilicate Ratio Total Volume:	
в.	Free-Standing Water Analysis	
	Volume of Decanted Water:	
	Volume of Sample:	
c.	Solidification Acceptability	
	 Percent of free-standing wate must be less than disposal factors 	er calculated in Section III.B (abov acility criteria.
	 Visual physical appearance: maintain its shape if removed 	Verify that the solidified waste wo d from the container.
		lts here)
		Date:
Bat	ch Solidification	
Α.	Formula (Per Drum Basis)	
	Waste to be solidified (gallons)	
в.		
в.	Sodium Metasilicate (added)	

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"PCP" Date February 28, 1984

TESTS FOR PORTLAND CEMENT

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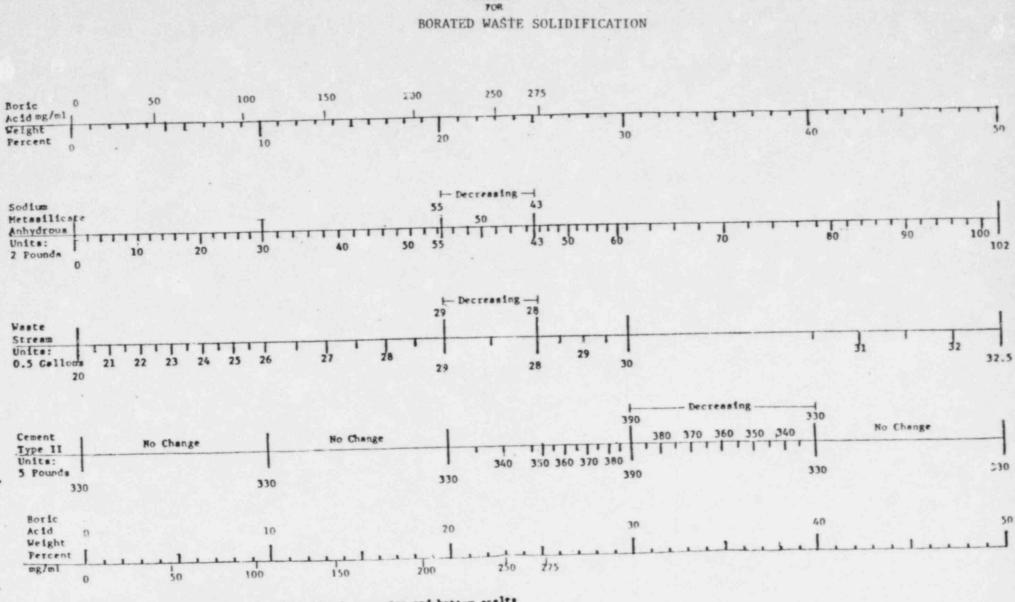
1.	Appearance (color)	I Light Greyish brown		III Light wn Greyish brown
2.	Median Particle Size (Microns)	200	300	60
3.	Consistency at			
	<pre>water ÷ cement ≈ 0.45 water ÷ cement = 0.3</pre>	Beef gravy Thick mortar		Stiff mortar Dry, crumbly
4.	Cup (Loose) Density (gm/cm ³) (ASTM 488-78)	1.16	1.04	0.93
5.	Formulation (by weight) 33% of 12% boric acid (pH=10) + 58% cement + 9% sodium carbonate			
	Consistency	Thin crack filler	Thick beef gravy	Soupy mortar
	Set time	4 days	1 hour	10 minutes
6.	Chemical Test - Example:			
	Tricalcium Aluminate Dicalcium Silicate	7 - 14% 15 - 24%	4 - 7% 23 - 33%	7 - 14% 10 - 22%

ATTACHMENT 2 Page 1 of 1

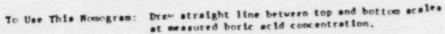
SPENT RESIN SOLIDIFICATION

Percent Free Standing Water	Pounds of Type III Cement	Gallons of Resins Slurry
10%	240 lbs	40.0 gals
15%	275 lbs	39.0 gals
20%	250 1bs	39.0 gals

A typical solidification would be to add two-thirds $(^{2}/_{3})$ of the waste on the first fill, tumble for two (2) minutes, the remainder of the volume on the second fill, followed by an eight (8) minute tumble.



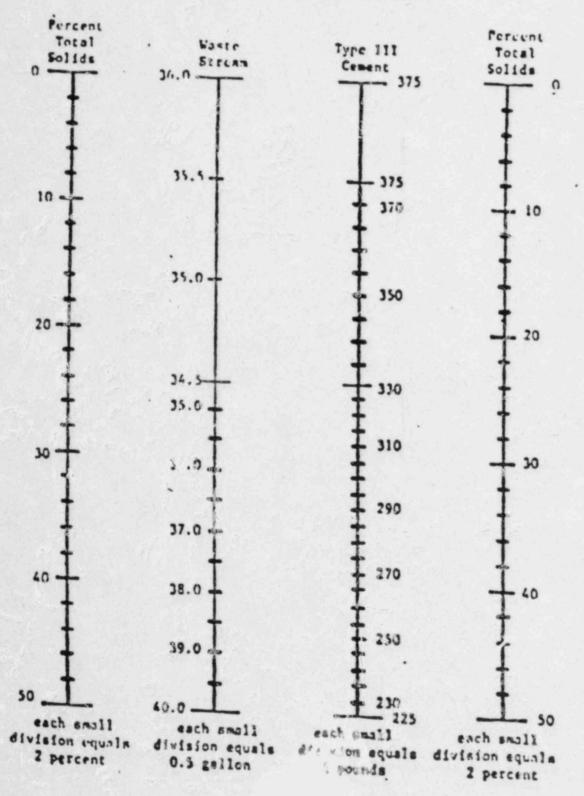
NOHOGRAM



Attachment 4 Page 1 of 1

NOMOGRAM FOR

NON-BORATED WASTE SOLIDIFICATION



UNION ELECTRIC COMPANY 1901 GRATIOT STREET ST. LOUIS, MISSOURI

DONALD F. SCHNELL

March 8, 1984

MAILING ADDRESS: P. O. BOX 149 ST. LOUIS, MISSOURI 63166

Mr. Harold R. Denton, Director Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Dear Mr. Denton:

ULNRC- 759

DOCKET NUMBER 50-483 CALLAWAY PLANT, UNIT 1 PROCESS CONTROL PROGRAM

Reference: ULNRC-694 dated December 2, 1983

The referenced letter transmitted the Process Control Program for Callaway Plant, Unit 1. Comments were generated as a result of NRC review of this submittal and informally given to UE on February 23, 1984. As a result of those comments, the Process Control Program has been revised and five copies are provided as an enclosure to this letter.

If there are any questions, please contact us.

Very truly yours,

Donald F. Schnell

DS/1w

Enclosure (5 copies)

STATE OF MISSOURI)) S S CITY OF ST. LOUIS)

Robert J. Schukai, of lawful age, being first duly sworn upon oath says that he is General Manager-Engineering (Nuclear) for Union Electric Company; that he has read the foregoing document and knows the content thereof; that he has executed the same for and on behalf of said company with full power and authority to do so; and that the facts therein stated are true and correct to the best of his knowledge, information and belief.

By Robert J. Schukai

General Manager-Engineering Nuclear

SUBSCRIBED and sworn to before me this 8th day of March, 1984.

BARBARA J. PFAFE

NOTARY PUBLIC STATE OF MISSOURI MY COMMISSION EXPIRES APRIL 22, 1985 ST. LOUIS COUNTY

cc: Glenn L. Koester Vice President Operations Kansas Gas & Electric P.O. Box 208 Wichita, Kansas 67201

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