October 20, 1983

Mr. Harold R. Denton, Director Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, DC 20555

> Subject: Quad Cities Station Units 1 and 2 NUREG 0737, Item II.D.1 - Relief and Safety Valve Test Requirements

NRC Docket Nos. 50-254 and 50-265

References (a): B. Rybak letter to H.R. Denton dated April 27, 1983.

(b): D.B. Vassallo letter to L.O. DelGeorge dated January 4, 1983.

Dear Mr. Denton:

As committed in Reference (a), an analysis of the Quad Cities Station safety relief valve discharge lines (SRVDLs) was performed in response to the Reference (b) NRC request for additional information. Two typical SRVDLs in each Quad Cities unit were analyzed to determine pipe stresses and support loads due to the dead weight of the water in the pipe, concurrent thermal effects, and for the effects of a postulated earthquake.

All pipe stresses and support loads were found to be within design allowables. Moments acting at the valve flanges during the alternate shutdown cooling mode of operation were found to be comparable to the design basis for steam discharge. The additional dead weight of the water for the alternate shutdown cooling mode of operation did not cause significant changes in deflections at the snubber locations, or significant changes in the load settings and operating ranges of the variable spring hangers. Based on the analysis of these lines, it is our judgment that the alternate shutdown cooling mode of operation can be readily accommodated for both units at Quad Cities Station.

If there are any questions concerning the above, please contact this office.

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H. R. Denton October 20, 1983 - 2 -One signed original and forty (40) copies of this letter are provided for your use. Very truly yours, B. Rybak Nuclear Licensing Administrator 1 m cc: NRC Resident Inspector - Quad Cities R. Bevan - NRR 7484N