

Georgia Power Company
333 Piedmont Avenue
Atlanta, Georgia 30308
Telephone 404 526-6526

Mailing Address:
Post Office Box 4545
Atlanta, Georgia 30302

L. T. Gucwa
Manager Nuclear Engineering
and Chief Nuclear Engineer



NED-84-040

February 10, 1984

Director of Nuclear Reactor Regulation
Attention: Mr. John F. Stolz, Chief
Operating Reactors Branch No. 4
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

NRC DOCKETS 50-321, 50-366
OPERATING LICENSES DPR-57, NPF-5
EDWIN I. HATCH NUCLEAR PLANT UNITS 1 AND 2
NUREG-0737 ITEM II.D.1 SAFETY RELIEF VALVE (SRV) TESTING

Gentlemen:

Your letter of December 29, 1982, requested Georgia Power Company to address questions related to the BWR Owners Group Safety Relief Valve (SRV) Test Program. Our response of April 4, 1983, stated that a supplemental response to a certain question (i.e., question #2) would be submitted to provide the conclusions of a dead weight analysis of the Hatch SRV discharge lines. We indicated in our letter of July 29, 1983, that no one single line could be identified as bounding, and therefore, unique analyses would be performed on each of the 22 SRV discharge lines.

The analyses included the weight and dynamic effects of water during an earthquake occurrence with the plant in the Alternate Shutdown Cooling Mode (ASCM) of operation. In this analysis, we have assumed a low probability for occurrence of an earthquake during the ASCM initiation transient since this initiation is done manually. Further, we have considered the dynamic effect of the initiation transient under service level B, while the earthquake event during the ASCM was considered under service level C. Hence, there is no need to combine the initiation transient loads with the earthquake loads.

A046
1/0

8403050085 840210
PDR ADOCK 05000321
P PDR

Director of Nuclear Reactor Regulation
Attention: Mr. John F. Stolz, Chief
Operating Reactors Branch No. 4
February 10, 1984
Page 2

Although the results of the analyses show that the type and orientation of supports and the piping geometry are adequate, we are still evaluating the strength of the piping supports. It is expected that modifications required to enhance the strength of supports, if any, will be minor in nature. Such modifications will be accomplished at a future outage when design and materials are available.

If you have any questions, please contact this office.

Very truly yours,

L. T. Gucwa

L. T. Gucwa

CT/lb

xc: H. C. Nix
J. P. O'Reilly - NRC Region II
Senior Resident Inspector