CONTROL BLOCK: |__|__|__|__|__|__| (1)   (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

| 0 | 1 | | W | I | P | B | H | 2 | (2) | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | (3) | 4 | 1 | 1 | 1 | 1 | (4) | | (5) |

LICENSEE CODE   LICENSE NUMBER   LICENSE TYPE   CAT

CON'T
| 0 | 1 | REPORT SOURCE | L | (6) | 0 | 5 | 0 | 0 | 0 | 3 | 0 | 1 | (7) | 0 | 8 | 1 | 7 | 8 | 3 | (8) | 1 | 0 | 0 | 5 | 8 | 3 | (9) |

NUMBER   EVENT DATE   REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0	2	During normal operation, inservice test IT-04 was conducted to determine
0	3	the operability of the RHR pumps. As a result of this test, RHR pump
0	4	2P10B was declared out of service due to high bearing vibration levels.
0	5	This placed the unit into a degraded mode of operation permitted by the
0	6	limiting condition defined in Technical Specification 15.3.3.A.3.a.
0	7	
0	8	

| 0 | 9 | SYSTEM CODE: R H (11) | CAUSE CODE: E (12) | CAUSE SUBCODE: B (13) | COMPONENT CODE: P U M P X X (14) | COMP. SUBCODE: B (15) | VALVE SUBCODE: Z (16) |

(17) LER/RO REPORT NUMBER | EVENT YEAR: 8 3 | SEQUENTIAL REPORT NO.: — 0 1 1 | OCCURRENCE CODE: / 0 3 | REPORT TYPE: X | REVISION NO.: — 1 |

ACTION TAKEN: A (18) | FUTURE ACTION: Z (19) | EFFECT ON PLANT: Z (20) | SHUTDOWN METHOD: Z (21) | HOURS (22): 0 0 0 0 | ATTACHMENT SUBMITTED: Y (23) | NPRD-4 FORM SUB.: Y (24) | PRIME COMP. SUPPLIER: N (25) | COMPONENT MANUFACTURER: P 0 2 5 (26) |

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	0	The cause of the high vibrations is uncertain at this time. Some
1	1	bearing roughness was noted upon disassembly of the pump. Corrective
1	2	action consisted of replacement of the failed pump.
1	3	
1	4	

FACILITY STATUS: E (23) | % POWER: 1 0 0 (29) | OTHER STATUS (30): N/A | METHOD OF DISCOVERY: B (31) | DISCOVERY DESCRIPTION (32): Routine inservice testing |

ACTIVITY RELEASED: Z (33) | CONTENT OF RELEASE: Z (34) | AMOUNT OF ACTIVITY (35): N/A | LOCATION OF RELEASE (36): N/A |

PERSONNEL EXPOSURES
| 1 | 7 | NUMBER: 0 0 0 (37) | TYPE: Z (38) | DESCRIPTION (39): N/A |

PERSONNEL INJURIES
| 1 | 8 | NUMBER: 0 0 0 (40) | DESCRIPTION (41): N/A |

LOSS OF OR DAMAGE TO FACILITY (43)
| 1 | 9 | TYPE: Z (42) | DESCRIPTION: N/A |

PUBLICITY (45)
| 2 | 0 | ISSUED: N (44) | DESCRIPTION: N/A |

NRC USE ONLY

NAME OF PREPARER   C. W. Fay   PHONE: 414/277-2811

ATTACHMENT TO LICENSEE EVENT REPORT NO. 83-011/03X-1

Wisconsin Electric Power Company
Point Beach Nuclear Plant Unit 2
Docket No. 50-301

On August 17, 1983, at 0414 hours, residual heat removal
(RHR) pump 2P10B was declared out of service due to high bearing
vibration levels. The out of service determination was made as
a result of data obtained during monthly inservice test IT-04.
This test procedure is based on ASME Code Section XI, Article
IWP-3000.

This event placed the unit into a degraded mode of operation
(one RHR pump available) which is permitted for 24 hours by the
limiting condition for operation defined in Technical Specification
15.3.3.A.3.a, and is reportable under Technical Specification
15.6.9.2.B.2.

RHR pump 2P10B was replaced on July 1, 1983 due to seal leakage.
This replacement took place during a Unit 2 refueling outage.
The pump which was removed was subsequently overhauled and placed
in stock as a spare.

The replacement pump was tested (per IT-04) from 1618 to
1757 hours on July 1, 1983, and showed vibration levels in the
alert range based on the baseline data for the previous pump.
Inservice test IT-04 was repeated on July 16, 1983 (2-week interval
based on alert range data) and all bearing vibration levels were
below the alert level based on baseline data for the previous
pump. It should be noted that the vibration instruments were
recalibrated during the 2-week interval. RHR pump 2P10B was retested
per IT-04 on August 16, 1983 (one month interval based on acceptable
range data) from 0359 to 0421 hours and showed horizontal vibration
levels in the action-required range on both pump bearings. A
retest was requested using different vibration monitors as allowed
by ASME Section XI IWP-3230(d). The retest was conducted from
0334 to 0356 hours on August 17, 1983. The vibration levels were
above the levels measured during the previous test and were all
in the action-required range. The auxiliary operator who conducted
the test stated that the pump seemed to be vibrating excessively
based on past experience. At 0414 hours on August 17, 1983, pump
2P10B was declared out of service. Inservice test IT-04 was com-
pleted at 0447 hours on 2P10A to ensure that 2P10A was available
while 2P10B was out of service. Maintenance personnel began work
on 2P10B after 2P10A had been tested. At 1650 hours on August 17,
1983, 2P10B was released by Maintenance for testing. The repair
consisted of replacing the pump with the now rebuilt pump which
was removed July 1, 1983. At 2050 hours on August 17, 1983, 2P10B
was declared back in service based on the successful completion
of IT-04.

The cause of the high vibrations is uncertain at this time.
Some bearing roughness was noted upon disassembly of the pump
but it is not known if this was the sole cause of the vibrations.
An evaluation is continuing to determine the cause(s) of the
high vibrations.

DMB

Wisconsin Electric POWER COMPANY
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

October 5, 1983

Mr. J. G. Keppler, Regional Administrator
Office of Inspection and Enforcement,
  Region III
U. S. NUCLEAR REGULATORY COMMISSION
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

DOCKET NO. 50-301
LICENSEE EVENT REPORT NO. 83-011/03X-1
RESIDUAL HEAT REMOVAL SYSTEM
POINT BEACH NUCLEAR PLANT, UNIT 2

Enclosed is Licensee Event Report No. 83-011/03X-1
(an updated report) with an attachment which provides a description
of an event reportable in accordance with Technical Specification
15.6.9.2.B.2, "Conditions leading to operation in a degraded mode
permitted by a limiting condition for operation or plant shutdown
required by a limiting condition for operation."

This Licensee Event Report was previously transmitted
to you by our letter dated September 16, 1983 under LER No.
83-009/03L-0; the correct number should have been 83-011/03L-0.
This updated report is being submitted to correct this error in
numbering and we apologize for any inconvenience we may have caused
you.

Very truly yours,

Vice President-Nuclear Power

C. W. Fay

Attachment

Copy to NRC Resident Inspector

OCT 11 1983
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