

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)										DOCKET NUMBER (2)					PAGE (3)		
Susquehanna Steam Electric Station - Unit 1										0 5 0 0 0 3 8 7					1 OF 2		

TITLE (4)

Core Alterations as a result of weekly surveillances.

EVENT DATE (6)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																								
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)																	
0	1	2	5	8	4	8	4	-	0	0	6	-	0	0	0	2	2	4	8	4						0	5	0	0	0			
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																																	
OPERATING MODE (9)		5	20.402(b)					20.406(c)					50.73(a)(2)(iv)					73.71(b)															
POWER LEVEL (10)		0	0	0	20.406(a)(1)(i)					50.36(c)(1)					50.73(a)(2)(v)					73.71(c)													
			20.406(a)(1)(ii)					50.36(c)(2)					50.73(a)(2)(vii)					OTHER (Specify in Abstract below and in Text, NRC Form 366A)															
			20.406(a)(1)(iii)				X	50.73(a)(2)(i)					50.73(a)(2)(viii)(A)																				
			20.406(a)(1)(iv)					50.73(a)(2)(ii)					50.73(a)(2)(viii)(B)																				
			20.406(a)(1)(v)					50.73(a)(2)(iii)					50.73(a)(2)(ix)																				

LISCENSEE CONTACT FOR THIS LOR (12)

NAME	TELEPHONE NUMBER	
L.A. Kuczynski - Nuclear Plant Specialist-III	AREA CODE	
	7 1 7	5 4 2 - 2 1 8 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)		X	NO	EXPECTED SUBMISSION DATE (15)			
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

It was determined that the performance of weekly SRM and IRM surveillances required with the Unit in Mode 5 is a CORE ALTERATION as defined by Technical Specification Definition 1.7. Such CORE ALTERATIONS are prohibited by various Technical Specifications. It was determined that the nuclear instrumentation movement required by the surveillances (approximately one inch) poses no safety hazard and that the surveillances should be performed as scheduled. A Technical Specification change is in process to provide relief from the movement restriction.

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PDR ADOCK 05000387
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-G104

EXPIRES: 8/31/85

FACILITY NAME (1) Susquehanna Steam Electric Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	— 0 0 6	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During the Unit 1/Unit 2 Tie-in Outage which commenced December 3, 1983, it was determined by the Shift Supervisor and Shift Technical Advisor on January 25, 1984, that the performance of weekly Source and Intermediate Range Monitor surveillances which require the movement of in-core instrumentation are CORE ALTERATIONS as defined by Technical Specification Definition 1.7. (The surveillances document the weekly channel functional test of the monitors.) Such movement is prohibited by Technical Specifications 3.6.5.1, 3.6.5.2, 3.6.5.3 and 3.7.2. It was determined that the nuclear instrumentation movement required by the surveillance, approximately one inch, presents no safety hazard and that the surveillances should be performed as scheduled. A Technical Specification change is in process to the Nuclear Regulatory Commission to provide relief from the restrictions posed by the above referenced Technical Specifications.



Pennsylvania Power & Light Company

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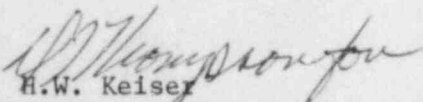
February 24, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 84-006-00
ER 100450 FILE 841-23
PLA-2102

Docket No. 50-387
License No. NPF-14

Attached is Licensee Event Report No. 84-006. This event is reportable per 10CFR50.73(a)(2)(i) in that, it was determined that the performance of weekly SRM and IRM surveillances required with the Unit in Mode 5 is a CORE ALTERATION as defined by Technical Specification Definition 1.7. Such CORE ALTERATIONS are prohibited by various Technical Specifications. Since the nuclear instrumentation movement required by the surveillances poses no safety hazard, a Technical Specification change is in process to provide relief from the movement restrictions.



H.W. Keiser
Superintendent of Plant-Susquehanna

LAK/cg

cc: Dr. Thomas E. Murley
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