NRC FOR	M 3,66			U. S. NUCLEAR REGULATORY COMMISSION
	· ·	LIC	CENSEE EVENT REPORT	
	CONTROL BLOCK:		() (PLEASE PRINT OR TYPE	ALL REQUIRED INFORMATION
	O H D B S 9 LICENSEE CODE	14 2 0 0 - 0	0 0 0 0 - 0 0 3 4 LICENSE NUMBER 25 26	LICENSE TYPE 30 4 57 CAT 58 5
CON'T 0 1 7 8	SOURCE 50	0 5 0 0 0 3	4 6 0 8 2 2 8 3 69 EVENT DATE	3 0 9 2 1 8 3 9 74 75 REPORT DATE 80
0 2	(NP-33-83-56)	During the perform	mance of ST 5061.02, the Co	ontainment Local Leak Rate
03	Test (LLRT),	four containment i	solation valves CC1407B, SA	2010, DR2012A, and CC1411B
04	were found wi	th leakage rates in	n excess of T.S. 3.6.1.2b a	and c. There was no danger
0 5	to the health	and safety of the	public or station personne	1. Two containment isola ion
06	valves are al	ways provided in s	eries. The backup isolatio	on valves for all four valves
07	showed essent	ially zero leakage	•	
08 7 8	9 5757	TEM CAUSE CAUS	E	COMP. VALVE
09		$ \begin{array}{c} D \\ D \\ 10 \end{array} $ $ \begin{array}{c} CODE \\ E \\ 11 \end{array} $ $ \begin{array}{c} CODE \\ E \\ 12 \end{array} $ $ \begin{array}{c} CODE \\ X \\ 12 \end{array} $	DE COMPONENT CODE 13 V A L V E X 14 13 18	
	17 REPORT	YEAR RE	PORTNO.	
	ACTION FUTURE TAKEN ACTION B 18 Z 19 33 34 34 CAUSE DESCRIPTION	22 23 24 EFFECT SHUTDOWN METHOD 2 35 20 2 36 N AND CORRECTIVE ACTION	26 27 ATTACHMENT NF HOURS 22 SUBMITTED FOR 37 40 41 23 4 NS 27	$\begin{array}{c cccc} & & & & & & \\ \hline RD-4 & & & & \\ \hline M & SUB. & & & \\ \hline Y & 24 & & & Z \\ 2 & & & 43 \end{array} \begin{array}{c} & & & & & \\ \hline Z & & & & \\ \hline 2 & & & & \\ \hline 3 & & & \\ \hline 2 & & & \\ 2 & & & \\ \hline 2 & & & \\ 2 & & & \\ \hline 2 & & & \\ 2 & & & \\ 2 & & & \\ 2 & & & \\ 2 & & \\ 2 & & & \\ 2 & & & \\ 2 & & & \\ 2 & & \\ 2 & & & \\ 2 & & \\ 2 & & & \\ 2$
10	The cause of	CC1407B, DR2012A,	and SA2010 failing their le	eak rate test was component
11	failures. DF	2012A had rust and	dirt on the seat. CC14071	B had a torn liner, and
1 2	SA2010 had di	rty and worn inter	mals. They have been repair	ired and retested satisfac-
13	torily. CC14	llB failed its lea	k rate test due to procedu	re deficiency. The valve
14	operator's to	orque limit switch	was out of adjustment due	to an inadequate procedure
15	D 28 0 0	OTHER STATU	US (30) METHOD OF DISCOVERY 44 B (31) During perf	DISCOVERY DESCRIPTION (32) formance of ST 5061.02
16 7 8	CTIVITY CONTENT ELEASED OF RELEASE	AMOUNT OF ACTIVIT	y (35) ↓ NA ↓ №	LOCATION OF RELEASE
1 7	NUMBER TY	BE DESCRIPTION (39)		
7 8	9 PERSONNEL INJURI	IS SCRIPTION (1) NA		
7 8	9 11 12 LOSS OF OR DAMAGE TO TYPE DESCRIPTION	FACILITY (43)	310120391 92000	Jezzil
7 8	9 10 PUBLICITY ISSUED DESCRIPTION	(45) P1	DR ADOCK 05000346 PDR	NRC USE ONLY
2 0 7 8	9 10 NA			68 69 80 5
DVR 83	-106 NAME OF P	REPARER Don Miss	ig/Ted Lang	PHONE: 419-259-5000, Ext. 254/535

TOLEDO EDISON COMPANY DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE SUPPLEMENTAL INFORMATION FOR LER NP-33-83-56

DATE OF EVENT: August 22, 1983

FACILITY: Davis-Besse Whit 1

IDENTIFICATION OF OCCURRENCE: Leakage of Containment Isolation Valves Found by Local Laak Rate Testing

Conditions Prior to Occurrence: The unit was in Modes 5 and 6, with Power (MWt) = 0 and Load (Gross MWe) = 0.

Description of Occurrence: During the performance of ST 5061.02, The Containment Local Leak Rate Test (LLRT), four containment isolation valves, CC1407B, SA2010, DR2012A, and CC1411B, were found with leakage rates in excess of Technical Specifications 3.6.1.2 (b and c).

Technical Specification 3.6.1.2.b limits combined leakage from type b and c tesus to less than 0.6La, approximately 600,000 standard cubic centimeters of air per minute (SCCM), and Technical Specification 3.6.1.2.c limits combined emergency ventilation "bypass leakage" to less than 0.015La, approximately 15,000 SCCM. Out of the above listed valves, only valves SA2010 and DR2012A fall under jurisdiction of Technical Specification 3.6.1.2.c. The action statement of Technical Specification 3.6.1.2 which requires restoration of leakage rates to within limits prior to increasing the Reactor Coolant System temperature above 200°F, has been met.

Designation of Apparent Cause of Occurrence: The cause of valves CC1407B, DR2012A, and SA2010 failing their leak rate test was component failure. Valve DR2012A had rust and dirt on the seat. Valve CC1407B had a torn liner. Valve SA2010 had dirty and worn internals.

The cause of CC1411B failing its leak rate test was due to procedure deficiency. Valve CC1411B had a limitorque operator with the torque limit switch out of adjustment, which caused the valve not to close to its seated position. This was caused by the adjustment procedure in MP 1410.32, Removal and Repair of Limitorque Valve Controls, being inadequate.

<u>Analysis of Occurrence:</u> There was no danger to the health and safety of the public or station personnel. Two containment isolation values are always provided in series. The backup (or second) isolation value for CC1407B, SA2010, DR2012A, and CC1411B all showed essentially zero leakage.

<u>Corrective Action:</u> Each of the valves listed above as being defective due to component failure have been repaired and retested satisfactorily. Procedure modification T-7430 was written for MP 1410.32 to improve the limitorque adjustment procedure. CC1411B was readjusted under procedure modification T-7430 and when it was retested, it had zero leakage. All other limitorque motor actuated valves were also retested if the limitorque actuator was worked on since the last local leak rate test was performed. TOLEDO EDISON COMPANY DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE SUPPLEMENTAL INFORMATION FOR LER NP-33-83-56 PAGE 2

SA2010 has been found to leak during previous leak rate tests and, therefore, the valve upstream will be isolated as well as SA2010 being maintained closed. The line, station air to containment, will also be blown down, and the valve's position, mentioned above, will be checked monthly.

Failure Data: Similar occurrences were reported in Licensee Event Reports NP-33-80-52 (80-042) and NP-33-82-31 (82-027).

LER #83-044

TOLEDO EDISON COMPANY

DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE SUPPLEMENTAL INFORMATION FOR LER NP-33-83-56 PAGE 3

VALVE	PENETRATION	VALVE DESCRIPTION	TECHNICAL SPECIFICATION*	LEAKAGE IN SCCM	CORRECTIVE MWO	RETEST DATE	RETEST LEAKAGE IN SCCM
CC1407B	4	Component cooling water return head- er containment outer isolation motor actuated butterfly valve	3.6.1.2.b	610,000	1-83-4392-00	9/3/83	2899
SA2010	42A	Station air to containment isola- tion valve	3.6.1.2.c	68,600	1-83-4600-00	9/6/83	736
DR2012A	13	Containment Sump Pump 1-1 discharge line - containment isolation valve - Control Room operated	3.6.1.2.c	350,000 (0 after stroking valve)	1-83-4629-00	9/15/83	0
CC1411B	3	Component cooling water supply header containment outer isolation motor actuated butterfly valve	3.6.1.2.b	Valve did not close to seated position	1-83-4693-01 1-83-4305-02	9/6/83	0

* Technical Specification 3.6.1.2.b limit equals approximately 600,000 SCCM Technical Specification 3.6.1.2.c limit equals approximately 15,000 SCCM

ATTACHMENT 1

NOTICE

1. . .

ATTACHED IS A RE-ISSUE OF LOG NO. K83-1320, LICENSEE EVENT REPORT NO. 83-044. THE PREVIOUS COPY HAD THE INCORRECT LER NUMBER, 83-043, ON THE TRANSMITTAL LETTER AND ON THE LICENSEE EVENT REPORT FORM.

PLEASE DESTROY ALL PREVIOUS COPIES OF THIS REPORT AND REPLACE WITH THE ATTACHED CORRECTED COPY.



September 21, 1983

Log No. K83-1320 File: RR2 (NP-33-83-56)

Docket No. 50-346 License No. NPF-3

Mr. James G. Keppler Regional Administrator, Region III Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

LER No. 83-044 Davis-Besse Nuclear Power Station Unit 1 Date of Occurrence: August 22, 1983

Enclosed are three copies of Licensee Event Report 83-043 which are being submitted in accordance with Technical Specification 6.9 to provide 30 day written notification of the subject occurrence.

Yours truly,

(ey D Muna

Terry D. Murray Station Superintendent Davis-Besse Nuclear Power Station

TDM/ljk

Enclosures

cc: Mr. Richard DeYoung, Director Office of Inspection and Enforcement Encl: 30 copies

> Mr, Norman Haller, Director Office of Management and Program Analysis Encl: 3 copies

Mr. Walt Rogers NRC Resident Inspector Encl: 1 copy

SEP 30 1983 FE221