(7-77)	LICENSEE EVENT REPORT
	CONTROL BLOCK:
	1 L Q A D 1 2 0 0 0 - 0 0 0 3 4 1
	REPORT LO 15 0 0 2 5 4 0 0 9 0 1 8 3 8 0 9 2 6 8 3 9 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80
0121	I On September 1, 1983, while preparing for Unit Two Refuel Outage, the Secondary
	Containment Canability Test was conducted in accordance with Technical Specifica-
	tion 4.7.6.1.c. The greatest vacuum attained was 0.15" of water. The consequences I
0 4	Cable and the greatest vacadum attained was only of water. The consequences
0 5	or this event are minimal because no core refueling operations were performed and
06	Secondary Containment integrity was maintained in accordance with Technical
0 7	Specification 3.7.C.1.
08	LB0
	SYSTEM CAUSE CAUSE CAUSE COMPONENT CODE SUBCODE SUBCODE SUBCODE SUBCODE SUBCODE SUBCODE SUBCODE SUBCODE
7 8	9 10 11 12 12 13 13 18 19 20 20 REVISION
	17 REPORT NO. CODE TYPE NO.
	ACTION FUTURE EFFECT SHUTDOWN HOURS 22 ATTACHMENT MPRD-4 PRIME COMPONENT MANUFACTURER A (18) Z (19) Z (20) Z (2) 0 0 0 V (23) N (24) Z (25) Z 9 9 9 (26)
	33 34 35 36 37 40 41 42 43 44 47 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
10	The cause of this occurrence was aged gaskets on various personnel access doors
1 1	entering the Reactor Building. The gaskets were replaced and the test conducted
1 2	satisfactorily on September 1, 1983.
13	
14	80
7 8	9 FACILITY STATUS SPOWER OTHER STATUS 30 METHOD OF DISCOVERY DESCRIPTION 32
1 5	E (28) 0 7 3 (29) NA B (31) ROULING SUPVENTIANCE 80
1 6 7 8	ELEASED OF RELEASE AMOUNT OF ACTIVITY (35) 3 3 Z 34 NA A4 45 LOCATION OF RELEASE (36) NA 80
17	PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39 NA 80
	PERSONNEL INJURIES NUMBER DESCRIPTION
7 8	9 11 12 LOSS OF OR DAMAGE TO FACILITY (12)
19	Z 42 NA
7 8	9 10 PUBLICITY PDR ADDCK 05000254 NRC USE ONLY
20	N (44) PDR 68 69 30. 5
	NAME OF PREPARER M Preuss PHONE 309-654-2241, ext 197

- 1. LER NUMBER: 83-32/03L-0
- 11. LICENSEE NAME: Commonwealth Edison Company Quad-Cities Nuclear Power Station
- III. FACILITY NAME: Unit One
- IV. DOCKET NUMBER: 050-254
- V. EVENT DESCRIPTION:

On September 1, 1983, with Unit One and Unit Two in the RUN mode at 595 MWe and 443 MWe, respectively, the Secondary Containment Capability Test was performed, as required by Technical Specification 4.7.C.l.c., before the upcoming Unit Two Refueling Outage. This Technical Specification requires to demonstrate the capability to maintain 0.25" of water vacuum on the Reactor Building with a Standby Gas Treatment flow of 4000 cfm prior to a refuel outage. The greatest vacuum obtained on this test was 0.15" of water.

It was determined that excessive in-leakage through access doors was the cause of the insufficient differential pressure. As a result, new gaskets were installed on all doors that were not sealed properly. The Secondary Containment Capability Test was successfully completed on September 1, 1983,; 0.25" of water vacuum was easily obtained. This is the first report on an incident of this type.

VI. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

Since the basis for this surveillance is to demonstrate Secondary Containment Capability prior to the time Primary Containment is opened, the consequences of this event are minimal. The refueling of Unit Two did not commence until after Secondary Containment Capability was fully demonstrated. Additionally, Secondary Containment integrity was maintained at all times during this incident, meeting Technical Specification 3.7.C.1.

VII. CAUSE:

The cause of this incident was excessive in-leakage to the Reactor Building through various access doors. The gasket material on these doors had deteriorated with age and afforded a poor seal. All personnel access doors, except one, leaked sufficiently to warrant replacement of the gaskets. The main trackway rail doors were identified as requiring gasket replacement.

VIII. CORRECTIVE ACTION:

The immediate corrective action taken was to replace the gasket material on all the poorly sealed personnel access doors and the main trackway doors. This proved effective. Action Item Report 4-83-13 has been initiated to investigate the feasibility of providing a more leak-tight seal on the interlock doors.



Commonwealth Edison Ovad Cities Nuclear Power Station 22710 206 Avenue North Cordova, Illinois 61242 Telephone 309/654-2241

NJK-83-338

September 26, 1983

J. Keppler, Regional Administrator Office of Inspection and Enforcement Region III U. S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, IL 60137

Reference: Quad-Cities Nuclear Power Station Docket Number 50-254, DPR-29, Unit One Appendix A, Sections 3.7.C. and 6.6.B.2.d

Enclosed please find Reportable Occurrence Report Number RC 83-32/03L-0 for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of Technical Specification 6.6.B.2.d; as an abnormal degradation of systems designed to contain radioactive material resulting from the fission process.

Respectfully,

COMMONWEALTH EDISON COMPANY QUAD-CITIES NUCLEAR POWER STATION

Referral

N. J. Kalivianakis Station Superintendent

NJK:JRW/bb

Enclosure

cc B. Rybak A. Morrongiello INPO Records Center

SEP 30 1983 IE22 .