NRC Form 366 (9-83)								LICENSEE EVENT REPORT (LER)						U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 315C-0104 EXPIRES: 8/31/85						
FACILITY	NAME	(1)	_				-							DOCKET NUMBER	(2)	PAGE (3)				
- nuisii			OF	RE	NUCL	EAR GE	NER	ATING	STAT	TON.	UNIT	2	-24.5	0  5   0   0	10131611	1 OF 012				
TITLE (4				-		27.11			-	20119	0.112.1			-1-1-1-	1-10-10-12	1-1-101-				
	JAN	JAR	Y :	16,	198	4, SPL	JRIO	US CO	NTAIN	MENT	PURGE	ISOL	ATION SIG	SNAL						
EVI	NT DAT	E (5)				LER NUMBE	R (6)		RE	POST DAT	TE (7)		OTHER	FACILITIES INVO	LVED (8)					
MONTH	DAY	AY YEA		YE				REVISION				FACILITY NAMES			DOCKET NUMBER(S)					
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		_	-	THE	S REPOR	T IS SUBMI	TTED PL	JRSUANT	TO THE R	EQUIREM	ENTS OF 10	CFR &	Check one or more	of the following) (1						
					20.402(b) 20.405(a)(1)(i)			E	20.405(e) 50.36(c)(1)			X 50.73(a)(2)(iv) 50.73(a)(2)(v)			73.71(b) 73.71(c)					
(10)	- 10	01010			20.405(a)(1)(ii) 20.405(a)(1)(iii) 20.405(a)(1)(iv) 20.405(a)(1)(iv)			50.38(e)(2) 50.73(e)(2)(i) 50.73(e)(2)(ii) 50.73(e)(2)(iii)			50.73(a)(2)(vii) 50.73(a)(2)(viii)(A) 60.73(a)(2)(viii)(B) 50.73(a)(2)(x)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
				_	10.400						FOR THIS	LER (12)	SS. FORTIER							
NAME												-			TELEPHONE NUM	BER				
													AREA CODE							
				J.	G. H	AYNES,	, ST	ATION	MANA	GER				7,1,4	419121-	17171010				
						COMPLE	TE ONE	LINE FOR	R EACH C	OMPONEN	T FAILURE	DESCRIBE	D IN THIS REPOR	RT (13)						
CAUSE	SYSTEM CO		ОМР	ONEN	NT MANUFAC- TURER			PORTABLE O NPRDS			CAUSE	SYSTEM COMPONENT		MANUFAC- TURER	REPORTABLE TO NPRDS					
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On 1/16/84, at 1050, with Unit 2 in Mode 5 and a routine main purge in progress, the Train "A" Containment Purge Isolation System (CPIS) was spuriously actuated from a spike on Containment Area Radiation Monitor 2RE7856. All Train "A" CPIS actuated valves closed to isolate the containment purge. Operators used redundant Containment Area Radiation Monitor 2RE7857 to verify that the actual containment area radiation levels were below the CPIS actuation setpoint. Train "A" CPIS and 2RE7856 were reset, and the containment purge was reinitiated.

At 1445, on 1/16/84, Train "A" CPIS was again spuriously actuated from a spike on 2RE7856, and the containment purge was again isolated. After verifying that actual containment radiation levels were below the CPIS actuation setpoint, the containment purge was reinitiated.

The spikes on 2RE7856 coincided with Train "A" Engineered Safety Features (ESF) subgroup relay testing. A voltage transient on the ESF bus, resulting from subgroup relay actuation, caused the sensitive gamma detector in 2RE7856 to spike, thereby initiating the CPIS. No corrective action is planned since minor voltage transients are expected during ESF subgroup relay testing. However, the containment purge was terminated until the completion of the Train "A" ESF testing.

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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

NRC Form 366A (9-83)	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION								U.S. NUCLEAR REGULATORY COMMISSIO APPROVED OMB NO. 3150-0104					
FACILITY NAME (1)			DOCKET NUMBER (2)		LE	R NUMBER (6)		PIRES 8/31	1/85 PAGE (3)					
	NUCLEAR	GENERATING STATION,		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER						
UNIT 2			0.15.10.10.10.13.6.1	8,4		0.0.4		0.0	0.2	OF	0.2			

On January 16, 1984, at 1050, with Unit 2 in Mode 5 and a routine containment main purge in progress, the Containment Purge Isolation System (EIIS Code VA) was spuriously actuated from a spike on Containment Area Radiation Monitor 2RE7856 (EIIS CODE RE). All Train "A" Containment Purge Isolation System actuated valves closed to isolate the containment purge. Operators used redundant Containment Area Radiation Monitor 2RE7857 to verify that the actual containment area radiation levels were below the Containment Purge Isolation System actuation setpoint. The Train "A" Containment Purge Isolation Signal and 2RE7856 were reset, and the containment purge was reinitiated.

At 1445, on January 16, 1984, the Train "A" Containment Purge Isolation System was again spuriously actuated from a spike on 2RE7856, and the containment purge was again isolated. After verifying that actual containment radiation levels were below the Containment Purge Isolation System actuation setpoint, the containment purge was reinitiated.

The first event was reported to the NRC pursuant to 10 CFR 50.72(b)(2)(ii). After the second event, operators identified the cause of these events and, therefore, did not consider another report appropriate or necessary. Our review of this matter has determined that the second event should have been reported pursuant to 10 CFR 50.72(b)(2)(ii). The need for supplementary training or instruction to operating personnel on the need to report ESF actuations pursuant to 10 CFR 50.72(b)(2)(ii) had been identified earlier (see LER 84-003, Docket 50-362), but corrective action had not been completed by January 16, 1984. Operators have been instructed to make all reports required by 10 CFR 50.72.

There are no credible circumstances under which this event would have been more severe.

Operators reviewed shift activities and determined that the spikes on 2RE7856 coincided with Train "A" ESF subgroup relay testing. A voltage transient on the ESF bus, resulting from subgroup relay actuation, caused the sensitive gamma detector in 2RE7856 to spike, thereby initiating the Containment Purge Isolation System. No corrective action is planned since minor voltage transients are expected during ESF subgroup relay testing. However, the containment purge was stopped at 2016 on January 16, 1984, until the completion of the Train "A" ESF testing.

## Southern California Edison Company



SAN ONOFRE NUCLEAR GENERATING STATION
P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES

TELEPHONE (714) 492-7700

February 15, 1984

U. S. Nuclear Ragulatory Commission Document Control Desk Washington, D.C. 20555

Subject: Docket No. 50-361 30-Day Report

July Report

Licensee Event Report No. 84-004

San Onofre Nuclear Generating Station, Unit 2

Pursuant to 10 CFR 50.73(a)(2)(iv), this submittal provides the required 30-day written Licensee Event Report (LER) for two occurrences involving the actuation of the Containment Purge Isolation System. Since these events involved the same components, system, cause and method of discovery, these events have been combined into a single report in accordance with NUREG-1022. The health and safety of plant personnel or the public were not affected by this event.

If you require any additional information, please so advise.

V6 Laynes

Enclosure: LER No. 84-004

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

U. S. Nuclear Regulatory Commission Office of Inspection and Enforcement

Institute of Nuclear Power Operations (INPO)

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