

CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)01 | C | A | S | 0 | S | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 58

CONT

01 | L | 6 | 0 | 5 | 0 | 0 | 0 | 3 | 6 | 1 | 7 | 0 | 8 | 2 | 3 | 8 | 3 | 8 | 0 | 9 | 2 | 3 | 8 | 3 | 9
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | With Unit 2 in Mode 1 and the Core Operating Limit Supervisory System (COLSS)

03 | out of service, calculations using Surveillance Test S023-3-3.6 indicated that

04 | the Departure from Nucleate Boiling Ratio (DNBR) margin was not within allowable

05 | limits. At 1900 the Action Statement associated with Limiting Condition

06 | for Operation (LCO) 3.2.4 was entered. This also occurred 9/1/83 at 0015,

07 | 9/2/83 at 1218, 9/4/83 at 1536, and 9/8/83 at 0846 and 1011. Public

08 | health and safety were not affected.

09 | I | A | 11 | X | 12 | X | 13 | X | X | X | X | X | X | 14 | X | 15 | Z | 16

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

17 | 8 | 3 | 1 | 1 | 1 | 6 | 1 | 0 | 3 | L | 1 | 0 |

21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

LER/RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NFRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

X | 18 | X | 19 | Z | 20 | Z | 21 | 0 | 0 | 0 | 0 | Y | 23 | N | 24 | N | 25 | Z | 9 | 9 | 9 |

33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | In each case, acceptable DNBR margin was verified within 15 minutes by restoring

11 | the COLSS to service. Cause and corrective action to prevent recurrence

12 | are under review and will be reported in a revision to this LER by November 15,

13 | 1983.

14 |

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)

15 | E | 28 | 1 | 0 | 0 | 29 | NA | A | 31 | Operator Observation

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

16 | Z | 33 | Z | 34 | NA | NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)

17 | 0 | 0 | 0 | 37 | Z | 38 | NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

PERSONNEL INJURIES NUMBER DESCRIPTION (41)

18 | 0 | 0 | 0 | 40 | NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)

19 | Z | 42 | NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

PUBLICITY ISSUED DESCRIPTION (45)

20 | N | 44 | NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

NAME OF PREPARER H. B. RAY

PHONE: 714/492-7700

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

H. B. RAY
STATION MANAGER

September 23, 1983

RECEIVED
NTC

SEP 27 PM 1:36

SCE

REGION V

TELEPHONE
(714) 492-7700

U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Maria Lane, Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. J. B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 83-116
San Onofre Nuclear Generating Station, Unit 2

Pursuant to Section 6.9.1.13.b of Appendix A, Technical Specifications to Facility Operating License NPF-10 for San Onofre Unit 2, this submittal provides the required 30-day written report and a copy of the Licensee Event Report (LER) form for six occurrences involving Limiting Condition for Operation (LCO) 3.2.4 associated with the Departure from Nucleate Boiling Ratio (DNBR) margin. These events have been combined into a single report in accordance with NUREG-0161.

On August 23, 1983 at 1900, with Unit 2 in Mode 1 and as a result of the Core Operating Limit Supervisory System (COLSS) being out of service, DNBR margin was determined in accordance with Procedure SO23-3-3.6. The DNBR margin determined in this manner was outside the region of acceptable operation as specified in Figure 3.2-1, "DNBR Margin Operating Limit Based on COLSS." As required by the Action Statement associated with Technical Specification 3.2.4, DNBR margin was determined to be within allowable limits within 15 minutes by returning COLSS to service and eliminating the need to rely on the more conservative DNBR margin calculated in accordance with SO23-3-3.6.

IE-22

September 23, 1983

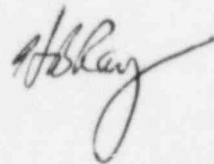
There was no impact on the health and safety of plant personnel or the public because of this event.

DNBR margin was determined to be outside allowable limits during similar periods of COLSS inoperability on September 1, 1983 at 0015; September 2, 1983 at 1218; September 4, 1983 at 1536; September 8, 1983 at 0846; and September 8, 1983 at 1011. For each such occurrence, acceptable DNBR margin was verified within 15 minutes by restoring the COLSS to service.

Corrective action to improve the reliability of COLSS is currently under review. This review will be completed by November 15, 1983 at which time LER 83-116 will be revised to indicate cause and corrective action.

If there are any questions regarding these events, please contact me.

Sincerely,



Enclosure: LER No. 83-116

cc: A. E. Chaffee (USNRC Resident Inspector, Units 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement

U. S. Nuclear Regulatory Commission
Division of Technical Information and Document Control

Institute of Nuclear Power Operations (INPO)