APPENDIX B

U. S. NUCLEAR REGULATORY COMMISSION

REGION IV

NRC Inspection Report: 50-285/83-35 License: DPR-40

Docket: 50-285

Licensee: Omaha Public Power District
1623 Harney Street
Omaha, Nebraska 68102

Facility Name: Fort Calhoun Station

Inspection At: Fort Calhoun Station, Blair, Nebraska

Inspection Conducted: November 1-30, 1983

Inspector: L A Yandell 12/9/83
L. A. Yandell, Senior Resident Reactor Inspector Date

Approved: W D Johnson 12/17/83
W. D. Johnson, Chief, Reactor Project Section C Date

Inspection Summary

Inspection Conducted November 1-30, 1983 (50-285/83-35)

Areas Inspected: Routine, announced inspection of operational safety verification, surveillance testing, and maintenance activities. The inspection involved 78 inspector-hours onsite by one NRC inspector.

Results: Within the three areas inspected, one violation was identified (violation - failure to follow procedures - paragraph 2).

DETAILS

1. Persons Contacted

*W. C. Jones, Division Manager, Production Operations
*W. G. Gates, Manager, Fort Calhoun Station
J. J. Fisicaro, Supervisor, Adminstrative Services & Security
L. T. Kusek, Supervisor, Operations
A. W. Richard, Supervisor, Technical
M. E. Eidem Jr., Manager, Mechanical Engineering
J. F. Ressler, Test Engineer
K. Gronberg, Quality Control Inspector
R. Wentworth, Quality Control Inspector

*Denotes attendance at the exit interview.

The NRC inspector also talked with, and interviewed, other licensee employees during the inspection. These employees included licensed and unlicensed operators, craftsmen, engineers, and office personnel.

2. Operational Safety Verification

The NRC inspector performed activities as described below to ascertain that the facility is being maintained in conformance with regulatory requirements and that the licensee's management control system is effectively discharging its responsibilities for continued safe operation.

a. The NRC inspector made several control room observations to verify proper shift manning, operator adherence to approved procedures, adherence to selected Technical Specifications, and operability of the reactor protective system. Selected logs, records, recorder traces, annunciators, panel indications, and switch positions were reviewed to verify compliance with regulatory requirements. The licensee's equipment control was reviewed for proper implementation by reviewing the maintenance order and tag-out logs, and by verifying selected safety-related tag-outs. Several shift turnovers were observed and shift turnover sheets were reviewed during this inspection period.

b. The NRC inspector toured the plant at various times to assess plant and equipment conditions. The following items were observed during these tours:

. general plant conditions

. vital area barriers not degraded or appropriately manned by security personnel

. adherence to requirements of radiation work permits (RWPs)

- proper use of protective clothing and respirators
- plant housekeeping and cleanliness practices including fire hazards and the control of combustible material
- work activities being performed in accordance with approved activities
- physical security
- HP instrumentation is operable and calibrated

c. The NRC inspector verified operability of the following safety-related systems by performing a walkdown and switch verification of the accessible portions of the system:

- High Pressure Safety Injection System per Checklist SI-1-CL-A
- Low Pressure Safety Injection System per Checklist SI-1-CL-B
- Plant Electrical Distribution - Normal Breaker Lineup per Checklist EE-1-CL-A
- Auxiliary Feedwater System per Checklist ST-FW-1-CL-A

The NRC inspector performed an indepth review of the auxiliary feedwater system to confirm that the lineup procedures agree with Plant Drawings 11405-M-253, Rev. 27 and 11405-M-254, Rev. 31, and the as-built configuration. Accessible portions of the system were reviewed for housekeeping practices. Hangers and supports were inspected and compared against OPPD CQE Piping Isometric Drawings D-4109, D-4238, Sheets 1-7, D-4240, Sheets 1 and 2, and D-4241. During this review, the NRC inspector identified on Drawing D-4238, Sheet 4 that Hydraulic Snubber FWS-72 was incorrectly shown on the drawing and identified in the data table when the device was no longer installed. The NRC inspector followed up this discrepancy and verified that the device was not required, and that support calculations substantiated the removal of this snubber. Although the physical installation is correct, the review and approval steps for this drawing were not performed properly in accordance with the requirements of Standing Order G-21, "Station Modification Control." Section 5.7.6 states that during post-installation checkout, the planner signs the Post-Installation Modification Control Form "to indicate that revisions to all modification planning documents have been properly reviewed and approved." This failure to adhere to the requirements of Standing Order G-21 is an apparent violation against Technical Specification 5.8.1 which requires that, "written procedures . . . be established, implemented, and maintained that meet or exceed the minimum requirements of

Sections 5.1 and 5.3 of ANSI N18.7-1972, and Appendix A of USNRC Regulatory Guide 1.33 . . .," (50-285/8335-01).

Checklist ST-FW-1-CL-A was compared with Checklist RC-2B-CL-D, "Reactor Startup Locked Valves," for consistency. Minor typographical errors were found on the CQE piping isometrics and were forwarded to the licensee for their information.

d. The NRC inspector reviewed Permit 83220 for "B" Monitor Tank and verified that the release was properly authorized, that the tank was recirced and sampled, and that the maximum release rate was calculated and established. It was noted that both radiological and chemical limits were calculated and satisfied, and the functional test on Discharge Valves HCV-691 and HCV-692 was performed using Detector RM-055A. The NRC inspector verified that the initial tank level and totalizer readings were recorded.

e. The NRC inspector observed portions of an emergency preparedness drill conducted by the licensee to exercise the emergency organizations in the control room/OSC, technical support center, and emergency offsite facility. A scenario was presented to provide experience at accident evaluation and dose assessment, and to test out the communications. Licensee observers were stationed at all the emergency facilities and a critique was held at the completion of the exercise.

No other violations or deviations were identified.

3. Surveillance Testing

The NRC inspector witnessed portions of the following surveillance test activities:

a. Diesel Generator Inspection (Annual) ST-ESF-6, F.5 and Diesel Generator Inspection-Instruments (Annual) ST-DG-2, F.1

b. Leakage Test of Waste Gas System (once each fuel cycle) ST-WG-1, F.6, for Waste Holdup Tanks WD-4A, WD-4B, and WD-4C. Minor typographical errors had to be corrected, and a procedure change was submitted to cover these items.

c. Station Batteries (Monthly) ST-DC-1, F.1. This was the repeat of a test performed previously which had shown that an equalizing charge was required.

d. Axial Power Distribution Channels (Monthly) ST-RPS-12, F.2

In the above surveillance tests, the NRC inspector verified, where applicable that:

- testing was scheduled in accordance with Technical Specification requirements
- procedures were being followed
- calibrated test equipment was being used
- qualified personnel were performing the tests
- limiting conditions for operation were being met
- test data were being accurately recorded

No violations or deviations were identified.

4. Maintenance Activities

The NRC inspector witnessed portions of the work performed on the following maintenance items:

a. Maintenance Order (MO) 22086, "Pressurizer Heater Bank." Backup Heater Group No. 9 failed to energize when required for a plant test. The NRC inspector reviewed the MO and verified that the document was properly routed and signed off, that appropriate Technical Specification references were made, and that QA/QC requirements were identified. The NRC inspector observed the work in progress and noted that a detailed work procedure, "Group 9 Pressurizer Heater Check," was PRC approved and attached to the MO. Drawing 11405-E-52, Sheet 8 was present at the job site, and a QC inspector was present to observe the check out of the breaker. A faulty relay was determined to be the problem and a qualified spare was installed. The NRC inspector verified that the QA material conformance tag was attached to the spare, and that the appropriate spare part information was recorded on the MO for use by the storeroom.

b. MO 22095, "Main Steam Isolation Valve Bypass Valve." HCV-1042-C was found to have a steam packing leak. The NRC inspector observed the packing being tightened and QC witnessed the work performed. Because of operating restrictions, Surveillance Test ST-ISI-MS-1, F.2 could not be performed but will be done during the next shutdown prior to exceeding 300°F. As a result, this MO will remain open until the test is performed and the valve packing repair proven acceptable.

c. MO 22327, "Steam Generator Level." During performance of Surveillance Test ST-ESF-11, F.1, it was found that steam generator level Instrument D/PIC-905 output meter jumped about 20 psia positive whenever the steam generator level setpoint signal was passed through. The setpoint was in specification, but maintenance work was performed

to correct this problem. The NRC inspector reviewed the MO for completeness and observed part of the work performed. The MO was signed off properly, QC verification was identified, qualified craftsmen performed the work, and an approved PRC procedure, "Investigate D/PIC-905 Sigma Jumping," was prepared and used. The equipment used was identifed and in calibration. Procedure Change 11255 was properly incorporated, and this expanded the troubleshooting effort into the power supply module when the readout device was proven operable. As part of this work the NRC inspector verified that Jumper 83-042 was installed properly and that a jumper log entry was made.

While working on the power supply module, Device D/102 dropped out, tripping "D" Channel RPS and causing a half-trip on Pressurizer Pressure Low Signal (PPLS). MO 22334 was issued to cover additional work to be performed, and the NRC inspector verified that the order was properly filled out and reviewed. A safety evaluation was completed, and a PRC approved procedure attached. Surveillance Tests ST-ESF-1, F.2; ST-ESF-11, F.1; ST-RPS-6, F.2; and ST-RPS-7, F.2 were performed after repairs were complete to verify operability.

d. SRDCO 83-54, "Spent Fuel Storage Rack Modification." The work has continued throughout this report period, and the three racks containing Boraflex have been installed at the south end of the spent fuel pool. In accordance with Amendment No. 75 to Facility Operating License No. DPR-40, the licensee has notified the NRC by Letter LIC-83-295, dated November 23, 1983, that the installation of the first new rack containing Boraflex was completed on November 19, 1983.

No violations or deviations were identified.

5. Exit Interview

The NRC inspector met with Messrs. Jones and Gates on December 2, 1983, to summarize the scope and findings of the inspection.