NRC Form 366 (9-83) LICENSEE EVENT REPORT (LER)									U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85							
FACILITY NAME (1)											DOCKET NUMBER (2)				AGE (3)	
Gra	nd Gu	1f N	iclea	r Station	- Unit	1	1			0 5 0 0	10	1 41 116	1 OF	0 12		
TITLE 14		A. A. 111	161.64	Dearton	CHIL	-					1-1-1-	1	1 11213	1 . 1	101-	
FSF	Actu	ation	ns Wh	ile Placi	no Batte	erv C	haroe	r on	Equal:	ize						
EVENT DATE (6)			15 WII	s While Placing Batte			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR SEQUENTIAL RE		REVISION	MONTH	DAY YEA	YEAR	FACILITY NAMES			DOCKET NUMBER(S)				
				NUMBER	NUMBER				NA		A	0	151010	101	1.1	
								1				-				
0 1	03	8 4	8 4	-loloh	00	012	0 2	8 4				0	151010	101	1 1	
-	BATING		-	ORT IS SUBMITTI	ED PURSUANT 1	TO THE R			CFR 8: /C	heck one or more	of the following) (1	-				
OPERATING MODE (8)			20.402(b)			20.406(e)		X 50.73(a)(2)(iv)		73.71(b)						
POWER			20.	20.406(a)(1)(i)			50.36(e)(1)			50.73(e)(2)(v)			73.71(e)			
(10) 0 010			20.	20.408(a)(1)(H)			50.38(c)(2)			50.73(a)(2)(vii)		OTHER (Specify in Abstract				
I W VI V			20.	20.408(a)(1)(iii)			50,73(a)(2)(i)			50.73(a)(2)(viii)(A)			below and in Text, NRC Form 366A)			
			20.	406(e)(1)(iv)		80.73(a)(2)(ii)			50,73(s)(2)(viii)	(8)					
			20.	20.406(a)(1)(v)			50.73(a)(2)(iii)			50.73(e)(2)(x)						
			-		1	ICENSEE	CONTACT	FOR THIS	LER (12)			-				
NAME												TEL	EPHONE NUM	8ER		
Ronald Byrd/Licensing Engineer								AREA CODE								
K	onald	Byr	1/L1C	ensing En	gineer						6 10 1 1	4	13 17 1-	12 11	1419	
				COMPLETE	ONE LINE FOR	EACH C	OMPONENT	FAILURE	DESCRIBE	D IN THIS REPO	PAT (13)	-				
				MANUFAC-	REPORTABLE						MANUFAC	REPORTABLE				
CAUSE	SYSTEM COMPO		TURER		TO NPRDS		CAUSE		SYSTEM COMPONEN		TURER		TO NPROS			
		1	1 1	1.1.1						1 1 1	1111					
10000																
	1	1		1.1.1.						1 1 1	111					
				SUPPLEM	ENTAL REPORT	EXPECT	ED (14)						MONTH	DAY	YEAR	
										SUBMISSION DATE (15)						
YE	S (If yes, co	omplete E	XPECTED	SUBMISSION DAT	E)		X NO				DATE	(6)	1	1	1	
ARSTRAC	T (Limit t	to 1400 so	aces (a a	proximetely fifteer	single space tun	curitian li	nes/ (18)					-				

On January 3, 1984, while in Cold Shutdown at 0920 hours and while placing a Division 2 battery charger on equalize, the Division 2 power supply tripped on high voltage resulting in the following automatic actions: initiation of Control Room Fresh Air Unit B, SGTS B, Drywell Purge Compressor B, SSW B, Division 2 hydrogen analyzers, Low Pressure Coolant Injection B and C, and isolation of Shutdown Cooling, RWCU, the Auxiliary Building and Containment Building. The Division 2 Diesel Generator was out of service at the time. The LPCI injection raised the water level to greater than 400 inches. While troubleshooting the problem on January 6, 1984, CRFAU B actuated when the chlorine detector was deenergized from a repeat of the trip. Other systems had been removed from service for the test.

The equalizing potentiometer on the battery charger was set higher than its normal equalizing voltage of 140 Vdc. The inverter tripped at 147 Vdc. The charger then tripped at 152 Vdc allowing the inverter to reset and initiate the ECCS actuation.

The procedure was revised to instruct the technicians to adjust the charger output to 140+/-1 Vdc when placing the chargers on equalize. A design change will lower the charger high voltage trip to 145 Vdc, allowing the charger to trip prior to the inverter trip.

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U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT (LER) TEXT CONTINUATION APPROVED OMB NO 3150-0104 EXPIRES 8/31/85 FACILITY NAME (1) DOCKET NUMBER (2) LER NUMBER (8) PAGE (3) SEQUENTIAL NUMBER YEAR REVISION NUMBER Grand Gulf Nuclear Station - Unit 1 0 |5 | 0 | 0 | 0 | 4 | 1 | 6 | 8 | 4 0 C | 1 012 OF 012 010 TEXT (If more specs is required, use additional NRC Form 366A's) (17)

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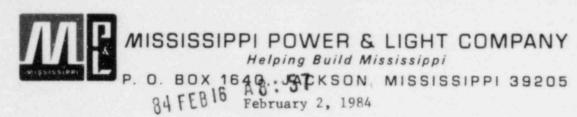
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The following additional information describes characteristics of the plant design which although not the root cause of the event contributed to the ECCS injection:

Rosemount Transmitter/Trip units are used in several General Electric supplied systems including the ECCS. On a low level in the reactor vessel, the current in the loop connecting the transmitter and the trip unit is below the specified setpoint. Consequently, the trip unit energizes the trip output load relay to initiate the ECCS.

On a loss of the 125 Vdc divisional power or 24 Vdc power to the trip unit, the loop current is zero and there is also no available power to energize the load relay. However, as soon as power is restored to the trip unit, power will be available to (and does) energize the load relay until the loop current re-establishes itself at a level above the trip setpoint. Even though the loop current restoration time is in the milliseconds range it is long enough for the trip to generate a trip output and to seal in the auto start logic of the ECCS. This situation caused the ECCS injection.

GGNS and General Electric are currently pursuing a design enhancement which will re-establish loop current prior to energization of the load relays to prevent inadvertent ECCS actuations following a loss of power to the trip unit.



NUCLEAR PRODUCTION DEPARTMENT

Document Control Desk U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Gentlemen:

SUBJECT: Grand Gulf Nuclear Station
Unit 1
Docket No. 50-416
License No. NPF-13
File 0260/L-835.0
ESF Actuations While Placing
Battery Charger on Equalize
LER 84-001-0
AECM-84/0059

Attached is Licensee Event Report (LER) 84-001-0 which is a final report.

Yours truly,

S NHOBBE

L. F. Dale

Manager of Nuclear Services

EBS/SHH:sad Attachment

cc: Mr. J. B. Richard (w/a)
Mr. R. B. McGehee (w/o)
Mr. T. B. Conner (w/o)
Mr. G. B. Taylor (w/o)

Mr. Richard C. DeYoung, Director (w/a) Office of Inspection & Enforcement U. S. Nuclear Regulatory Commission Washington, D. C. 20555

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