

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: 1 2 3 4 5 6 7 8

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	7	8	A	R	A	N	0	12	14	0	0	-	0	0	0	0	0	-	0	0	13	4	1	1	1	1	14	57	CAT	58
LICENSEE CODE				LICENSE NUMBER																		LICENSE TYPE									
0	1	7	8	REPORT	L	16	0	5	0	0	0	3	6	8	17	0	8	1	8	8	3	18	0	9	1	9	8	3	19		
SOURCE				60		DOCKET NUMBER										EVENT DATE						REPORT DATE									
				61		68										69						74						75		80	

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

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On 8/18/83, while in Mode 1 at 24% full power (FP), Core Protection Calculator (CPC) Channel "A" was placed in "bypass" and Channel "D" was tripped thereby entering the action statement of Technical Specification (TS) 13.3.1.1. This action placed the CPCs in 1 out of 2 logic with Channels "B" and "C" operable and is reportable per TS 6.9.1.9.b. The above action was taken as a precautionary measure following observation that a CPC addressable constant (point identification number 064 - Thermal Power Calibration Constant) was reading less than the minimum unrestricted value allowed by the Technical Specifications without prior PSC approval. There were no potential adverse affects associated with this event since the values of the constant were subsequently

SYSTEM	CAUSE	EFFECT

[illegible]

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

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1	0	ICPC Channels "A" and "D" were removed from service because point identification number 064 for these channels
1	1	was found to be reading less than the minimum unrestricted allowable value without previous Plant Safety
1	2	Committee (PSC) review and concurrence. The affected constants had been inserted into the CPCs the previous
1	3	day. Following observation that the values were less than the unrestricted allowable value, the PSC reviewed
1	4	the calculated values and concurred that they were correct and appropriate, and the CPCs were returned to

FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
<u>1</u> <u>5</u> <u>E</u> 28	<u>0</u> <u>2</u> <u>4</u> 29	NA	30 <u>B</u> 31	Routine Surveillance
7 8	9 10 12	13	44 45	46 80

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
7	6 8	Z 33 9	NA 10	35 44
		Z 34 11		NA 45
				36 80

PERSONNEL EXPOSURES									
NUMBER		TYPE		DESCRIPTION					
1	7	0	0	0	37	2	38	NA	139
8	9	11	12	13					80

PERSONNEL INJURIES										
NUMBER		DESCRIPTION								
1	8	1	0	0	0	40	NA			
8		9	11			12				
		LOSS OF 22 INJURED								
		80								

LOSS OF OR DAMAGE TO FACILITY	
TYPE	DESCRIPTION
1 9	1 2 42 NA

[illegible]

NAME OF PREPARER: Patrick Rogers

PHONE: (501) 964-3100

LICENSEE EVENT REPORT

EXHIBIT A

LER No. 50-368/83-038/03L-0

Occurrence Date: 08/18/83

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (Continued)

reviewed by the PSC and found to be correct.

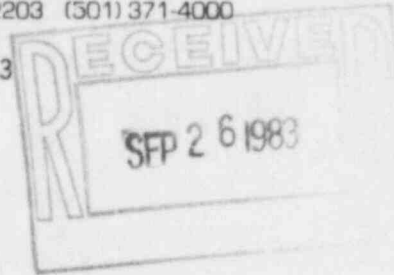
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (Continued)

normal status the same day. In order to prevent recurrence, related procedures have been revised to require specific comparison of calculated values to TS limits and signoff provisions have been added to the procedures. Also, clarification has been made to better define the differences between software limits and TS limits.



ARKANSAS POWER & LIGHT COMPANY
POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000

September 19, 1983



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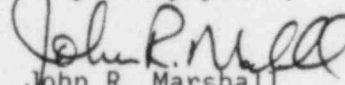
Mr. W. C. Seidle, Chief
Reactor Project Branch #2
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

Subject: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Licensee Event Report
No. 83-038/03L-0

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 2 Technical Specification 6.9.1.9.b, attached is the subject report concerning problems with the Core Protection Calculator Channels "A" and "D".

Very truly yours,


John R. Marshall
Manager, Licensing

JRM:RJS:s1

Attachment

cc: Mr. Richard C. DeYoung
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. Norman M. Haller, Director
Office of Management & Program Analysis
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

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