

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

A horizontal number line with vertical tick marks at intervals of 1. The first tick mark is labeled '1' and the sixth tick mark is labeled '6'. To the right of the '6' tick mark is a circled number '1'.

CON'T

0	1
7	8

REPORT SOURCE

L	6	0	5	0	0	0	3	6	6	7	0	8	2	3	8	3	8	0	9	2	2	8	3	9
60	61									68	69						74	75						80
DOCKET NUMBER											EVENT DATE						REPORT DATE							

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 While performing HNP-2-1119 "ECCS STATUS CHECK" procedure, plant
0 3 personnel discovered the 2E11-F007B (RHR "B" LOOP MINIMUM FLOW
0 4 VALVE) had no position indication. The "B" Loop of RHR was in the torus
0 5 cooling mode at this time and the 2E11-F007B valve would normally be
0 6 closed. This event is contrary to the requirements of T.S. 3.5.3.2.
0 7 The "A" RHR and the "A" and "B" CSS subsystems were operable. The health
0 8 and safety of the public were not affected by this non-repetitive event.

SYSTEM CODE C F		CAUSE CODE E		CAUSE SUBCODE A		COMPONENT CODE X X X X X X						COMP. SUBCODE X		VALVE SUBCODE Z			
9 10		11 12		12 13		13 14 15 16 17 18						19 20		21 22			
EVENT YEAR 8 3		SEQUENTIAL REPORT NO. 0 6 0		OCCURRENCE CODE 0 3		REPORT TYPE L		REVISION NO. 0									
21 22		23 24		25 26		27 28		29 30		31 32							
ACTION TAKEN A		FUTURE ACTION Z		EFFECT ON PLANT Z		SHUTDOWN METHOD Z		HOURS 0 0 0 0		ATTACHMENT SUBMITTED Y		NPRD-4 FORM SUB. N		PRIME COMP. SUPPLIER L		COMPONENT MANUFACTURER M 1 7 5	
33 34		35 36		37 38		39 40		41 42		43 44		45 46		47 48		49 50	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of this event was a blown fuse in the control
1 1 circuit of the 2E11-F007B valve. The fuse blew due to a loose fuse
1 2 holder. The fuse holder was tightened and the fuse was replaced.
1 3 2E11-F007B was satisfactorily functionally tested per HNP-2-3164 "RHR
VALVE OPERABILITY" procedure and returned to operable status on 8/24/83.

8 9
FACILITY STATUS (28) % POWER (0 0 8) (29) OTHER STATUS (30) NA
7 8 9 10 11 12 13 14 44
45 46 80
METHOD OF DISCOVERY (31) B Operator Observation (32)
7 8 9 10 11 12 13 14 44
45 46 80
ACTIVITY CONTENT (33) Z (34) Z (35) NA
7 8 9 10 11 12 13 14 44
45 46 80
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) NA LOCATION OF RELEASE (36)
7 8 9 10 11 12 13 14 44
45 46 80
PERSONNEL EXPOSURES (37) 0 (38) Z (39) NA
7 8 9 10 11 12 13 14 44
45 46 80
NUMBER TYPE DESCRIPTION (39) NA
7 8 9 10 11 12 13 14 44
45 46 80
PERSONNEL INJURIES (40) 0 (41) 0 (42) 0 (43) NA
7 8 9 10 11 12 13 14 44
45 46 80
NUMBER DESCRIPTION (41) NA
7 8 9 10 11 12 13 14 44
45 46 80
LOSS OF OR DAMAGE TO FACILITY (43) NA
7 8 9 10 11 12 13 14 44
45 46 80
TYPE DESCRIPTION (43) NA
7 8 9 10 11 12 13 14 44
45 46 80
PUBLCITY (45) NA
7 8 9 10 11 12 13 14 44
45 46 80
ISSUED (44) N (45) NA
7 8 9 10 11 12 13 14 44
45 46 80
8310030486 830922
PDR ADOCK 05000366
S PDR
IE22
NRC USE ONLY
68 69 80

NAME OF PREPARER S. B. Tipps

PHONE:

(912) 367-7851

NARRATIVE REPORT
FOR LER 50-366/1983-060

LICENSEE : GEORGIA POWER COMPANY
FACILITY NAME : EDWIN I. HATCH
DOCKET NUMBER : 50-366

Tech. Specs. section(s) which requires report:

This 30-day LER is required by Tech. Specs. section 6.9.1.9.b due to the event's showing that the unit was not meeting the requirements of Tech. Specs. section 3.5.3.2.

Plant conditions at the time of the event(s):

This event occurred on 08/23/83 with the reactor mode switch in the run position and reactor power at 200 MWt (approximately 8% power).

Detailed description of the event(s):

On 08/23/83 while performing the "ECCS STATUS CHECK" procedure (HNP-2-1119), plant personnel discovered the 2E11-FO07B (RHR B Loop Minimum Flow Valve) had no position indication. The "E" Loop of RHR was in the torus cooling mode at this time. (Note: with the "B" Loop of RHR in operation the minimum flow valve 2E11-FO07B would normally be closed.)

Consequences of the event(s):

Plant operation was not affected by this event. The health and safety of the public were not affected by this event.

Status of redundant or backup subsystems and/or systems:

There is no backup or redundant system for the 2E11-FO07B valve. The "A" and "B" Core Spray subsystems were operable. The "A" RHR subsystem was operable.

Justification for continued operation:

Unit 2 was placed in a 4 hour LCO as required by Tech. Specs. section 3.6.3. However, with the "B" Loop of RHR in the torus cooling mode the 2E11-FO07B valve was closed and the requirements of Tech. Specs. section 3.6.3 were satisfied. A 7-day LCO should have been entered under Tech. Specs. section 3.5.3.2, ACTION a.1.

If repetitive, number of previous LER:

This event is non-repetitive.

Narrative Report for LER 50-366/1983-060
Page Two

Impact to other systems and/or Unit:

This event did not impact any other systems on Unit 2 or Unit 1.

Cause(s) of the event(s):

The cause of this event is component failure. A blown fuse was found in the control circuit of the 2E11-F007B valve. The fuse blew due to a loose fuse holder.

Immediate Corrective Action:

The fuse holder was tightened. and the fuse was replaced. 2E11-F007B was satisfactorily functionally tested per the "RHR VALVE OPERABILITY" procedure (HNP-2-3164) and returned to operable status on 08/24/83 (less than 2 hours after the failure).

Supplemental Corrective Action:

None required.


Scheduled (future) corrective action:

None required.

Action to prevent recurrence (if different from corrective actions):

None required.

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83 SEP 26 4 8: 53
 Georgia Power

Edwin I. Hatch Nuclear Plant

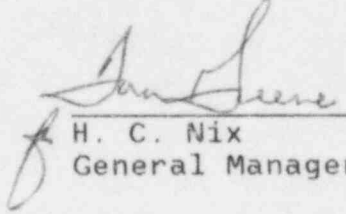
September 22, 1983
GM-83-939

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
Suite 3100
101 Marietta Street
Atlanta, Georgia 30303

ATTENTION: Mr. James P. O'Reilly

Attached is Licensee Event Report No. 50-366/1983-060. This report is required by Hatch Unit 2 Technical Specifications Section 6.9.1.9.b.


H. C. Nix
General Manager

SLC
HCN/SBT/djs

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