

LICENSEE EVENT REPORT

Attachment to AEEM-83/0606
Page 1 of 1

Page 1 of 1

CONTROL BLOCK:

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(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | M | S | G | G | S | 1 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 57 CAT 58 | 5

8 9 14 15 25 26 30 57 58

LICENSEE CODE LICENSE NUMBER LICENSE TYPE

CON'T

REPORT
SOURCE

L 6 0 5 0 0 0 4 1 6 7 0 8 2 2 8 3 8 0 9 2 1 8 3 9
 80 81 DOCKET NUMBER 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99
 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

0 2 On August 22, 1983, during surveillance testing, the CR Ventilation

0 3 Radiation Monitor channel B Hi and Hi-Hi and channel C Hi-Hi setpoints

0 4 were discovered to exceed the allowable values of T.S.3.3.7.1-1.6. Chan-

0 5 nel B Hi was set at 4.33 mR/hr vs an allowable of less than or equal to

0 6 4 mR/hr. Channel B Hi-Hi was set at 5.25 and channel C at 9.64 mR/hr vs

0 7 allowable of less than or equal to 5 mR/hr. There was no effect on the

0 8 health and safety of the public nor was their a threat to plant safety.

7 8 9

CAUSE VALVE

0 9		SYSTEM CODE B B (11)		CAUSE CODE E (12)		CAUSE SUBCODE E (13)		COMPONENT CODE I N S T R U (14)		SUBCODE S (15)		SUBCODE Z (16)	
7 8		9 10		11 12		13 14		15 16		17 18		19 20	
17 LER/RO REPORT NUMBER		EVENT YEAR B 3 (21) 22		23		SEQUENTIAL REPORT NO. 1 2 7 (24) 25		OCCURRENCE CODE / (27)		REPORT TYPE 0 3 (28) 29		L (30) 31	
ACTION TAKEN E (33) 34		FUTURE ACTION Z (35) 36		EFFECT ON PLANT Z (37) 38		SHUTDOWN METHOD Z (39) 40		HOURS 0 0 0 0 (41) 42		ATTACHMENT SUBMITTED N (43) 44		NPRD-1 FORM SUB. N (45) 46	
PRIME COMP. SUPPLIER N (47) 48		REVISION NO. 0 (49) 50		COMPONENT MANUFACTURER G 0 8 0 (51) 52		CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)							

CAUSE DESCRIPTION AND CORRECTIVE ACTION

1 0 The cause was attributed to instrument drift. The instruments were

1 1 recalibrated to within tolerances. This is reported pursuant to

1 2 I.S.6.9.1.13.a. This is a final report.

1 3

1 4
7 8 9
FACILITY STATUS
1 5 6 28
7 8 9 10 11 12 13
% POWER
0 0 0 0 28
OTHER STATUS 30
NA
METHOD OF DISCOVERY
8 31
Surveillance Testing
DISCOVERY DESCRIPTION 32
ACTIVITY CONTENT
RELEASED OF RELEASE
1 6 7 33
7 34
AMOUNT OF ACTIVITY 35
NA
LOCATION OF RELEASE 36
NA

PERSONNEL EXPOSURES									
NUMBER		TYPE		DESCRIPTION					
1	7	0	0	0	37	2	38	NA	39

PERSONNEL INJURIES		NUMBER		DESCRIPTION	
1	8	0	0	0	NA

LOSS OF OR DAMAGE TO FACILITY					
TYPE		DESCRIPTION			
1	9	7	(42) NA	8310030317 830921 PDR ADOCK 05000416 S PDR	<i>IE22</i>

7 8 9 10
PUBLICATION DESCRIPTION (45)
ISSUED (44) NA
7 8 9 10
68 69 70 71 72 73 74 75 76 77 78 79 80
NRC USE ONLY
TE 22

NAME OF PREPARER D. E. Cathey

PHONE: _____

NRC USE ONLY

IE 22

91 7-920



MISSISSIPPI POWER & LIGHT COMPANY

Helping Build Mississippi

P. O. BOX 1640, JACKSON, MISSISSIPPI 39205

September 21, 1983

83 SEP 26 4 8:49

NUCLEAR PRODUCTION DEPARTMENT

U.S. Nuclear Regulatory Commission
Region II
101 Marietta St., N.W., Suite 2900
Atlanta, Georgia 30303

Attention: Mr. J. P. O'Reilly, Regional Administrator

Dear Mr. O'Reilly:

SUBJECT: Grand Gulf Nuclear Station
Unit 1
Docket No. 50-416
License No. NPF-13
File: 0260/L-835.0
CR Ventilation Radiation Monitor
Exceeds Allowable Technical
Specification Values
LER 83-127/03 L-0
AECM-83/0606

On August 22, 1983, during surveillance testing, the Control Room Ventilation Radiation Monitor Setpoints were found to exceed the allowable values of Technical Specification 3.3.7.1-1.6. This is reported pursuant to Technical Specification 6.9.1.13.a. Attached is LER 83-127/03 L-0 which is a final report.

Yours truly,

L F Dale
for L. F. Dale
Manager of Nuclear Services

EBS/SHH:rg
Attachment

cc: Mr. J. B. Richard (w/a)
Mr. R. B. McGehee (w/o)
Mr. T. B. Conner (w/o)
Mr. G. B. Taylor (w/o)

Mr. Richard C. DeYoung, Director (w/a)
Office of Inspection & Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

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