NRC MONTHLY OPERATING REPORT

DOCKET NO 50-361 DATE 1 April 1982 COMPLETED BY T. Hanaford TELEPHONE 714-492-7700 Ext. 704

OPERATING STATUS

1. Unit Name: San Onofre, Unit 2 2. Reporting Period: 1 March 1982-31 3. Licensed Thermal Power (MWt): 169.5 4. Nameplate Rating (Gross MWe): 1127 M 5. Design Electrical Rating (Net MWe): 1087 6. Maximum Dependable Capacity (Gross MWe):				
7. Maximum Dependable Capacity (Net MWe): 8. If Changes Occur in Capacity Ratings (Items Nu N/A	nce Last Report, Give Rea	asons:		
9. Power Level To Which Restricted, If Any (Net 1)	MWe): N/A	startun nhase	o testing	
O. Reasons For Restrictions, If Any: Unit i	5 III MITCIAL	Startup phase	or testing	
	This Month	Yrto-Date	Cumulative	
l. Hours In Reporting Period	744	1056	1056	
. Number Of Hours Reactor Was Critical	0	0	0	
Reactor Reserve Shutdown Hours	0	0	0	
Hours Generator On-Line	0	0	0	
. Unit Reserve Shutdown Hours	0	0	0	
Gross Thermal Energy Generated (MWH)	0	0	0	
Gross Electrical Energy Generated (MWH)	0	Ú	0	
Net Electrical Energy Generated (MWH)	0	0	0	
Unit Service Factor	0	0	0	
. Unit Availability Factor	0	0	0	
Unit Capacity Factor (Using MDC Net)	0	0	0	
. Unit Capacity Factor (Using DER Net)	0	0	0	
. Unit Forced Outage Rate	0	0	0	
N/A Shutdowns Scheduled Over Next 6 Months (Type	e. Date, and Duration	of Each):		
. If Shut Down At End Of Report Period, Estimat	ed Date of Startup: .	N/A		
6. Units In Test Status (Prior to Commercial Operation):		Forecast	Achieved	
		E/22/02		
INITIAL CRITICALITY		<u>5/22/82</u> 7/11/82		
INITIAL ELECTRICITY	THE PERSON NAMED IN COLUMN 1	-		
COMMERCIAL OPERATION		8/15/82		

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DOCKET NO. 50-361 UNIT 2 DATE 4/1/82 COMPLETED BY T. Hanaford

			CIVIT
DAY	AVERAGE DAILY POWER LEVE		COMPLETED BY T. TELEPHONE 714- Ext.
2 3 4	0	DAY	
6 -	0 0	18 19 20	0
8 -	0	21 - 22 - 23	0 0
10 11 12	0	24	0
13	0	26 27 28	0 0
5	0	29	0 0 0
			0

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UNIT SHUTDOWNS AND POWER REDUCTIONS

50-361 DOCKET NO. UNIT NAME SONGS 2 1 April 1982 DATE COMPLETED BY I. Hanaford TELEPHONE 714-492-7700

REPORT MONTH March

Ext. 704 Method of Shutting Down Reactor³ Reason? Duration (Hours) Type Licensee Cause & Corrective Date Event Action to Report # Prevent Recurrence N/A N/A N/A N/A N/A N/A N/A N/A N/A N/A

Forced S. Scheduled

Reason

A-Equipment Failure (Explain) B-Maintenance or Test

C-Refueling

D Regulatory Restriction

1 Operator Training & License Examination

F-Administrative

G-Operational From (Explain)

H-Other (1 xplain)

Method:

I-Manual

2-Manual Scrain.

3-Automatic Scram.

4-Other (Explain)

Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee

Event Report (LER) File (NURLG 01611

Exhibit H. Same Source

SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

DOCKET NO. 50-361

UNIT 2 DATE 1 April 1982 COMPLETED BY T. Hanaford TELEPHONE 714-492-7700 Ext. 704 Date/Time Event March 1, 0001 Unit 2 in Mode 6, fuel alignment verification in progress March 13, 1400 NRC approved setting of Low Temperature Over Pressure Relief valve at 408 psig. Discontinued 12 hour verification of RCS vent path. March 14, 0525 Lost Shutdown Cooling flow March 14, 0557 Unusual Event declared due to Loss of Shutdown Cooling March 14, 0710 Shutdown cooling flow restored. March 14, 0722 Unusual Event terminated. March 14, 0915 RCS boron concentration is at 1962 ppm. Prior to event, concentration was at 2004 ppm. March 14, 1240 NRC notified of RCS dilution event March 17, 0425 Entered Mode 5. March 18, 0010 Notified by SCE NQC that a three foot stainless steel ruler was dropped into the reactor vessel through an ICC instrument nozzle. March 19, 0020 Entered Mode 6 March 19, 2037 Reactor vessel head lifted and metal ruler removed. March 20, 2330 Entered Mode 5.

Date/Time

Event

March 31, 2400

Unit 2 in Mode 5. RCS level 6 3/4" above flange, fill and vent in progress. RCS boron concentration is at 1875 ppm.

There were no challenges to the safety valves during this reporting period.

REFUELING INFORMATION

DCCKET NO. 50-361

	UNIT SONGS 2			
	DATE _1 April 1982			
	COMPLETED BY T. Hanaford			
	TELEPHONE <u>714-492-7700</u> Ext. 704			
1.	Scheduled date for next refueling shutdown.			
	Approximately 17 months after start of commercial operation			
2.	Scheduled date for restart following refueling.			
	Not yet determined			
3.	Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?			
	Not yet determined What will these be?			
	Not yet determined			
4.	Scheduled date for submitting proposed licensing action and supporting information.			
	Not yet determined			
5.	Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.			
	Not yet determined			
6.	The number of fuel assemblies.			
	a) In the core 217			
	b) In the spent fuel storage pool. 0			
7.	Licensed spent fuel storage capacity800			
	Intended change in spent fuel storage capacityN/A			
8.	Projected date of last refueling that can be discharged to spent fuel storage pool assuming present capacity.			
	Not yet determined			

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58 A)

Southern California Edison Company

SCE

SAN ONOFRE NUCLEAR GENERATING STATION
P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

H. B. RAY

TELEPHONE (714) 492-7700

Director
Office of Management Information and
Program Analysis
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Docket No. 50-361 San Onofre Unit No. 2

Gentlemen:

Enclosed is the data required by Technical Specification Section 6.9.1.10 for the San Onofre Nuclear Generating Station, Unit 2.

Please contact us if we can be of further assistance.

Sincerely,

Enclosures

cc: Director, Region V
Office of Inspection and Enforcement

A. Chaffee - USNRC Inspector, Unit 2

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