Docket Nos: 50-413

and 50-414

50-413

MEMORANDUM FOR: Atomic Safety and Licensing Board for Catawba Nuclear Station, Units 1 and 2

FROM:

Thomas M. Novak, Assistant Director

for Licensing

Division of Licensing

SUBJECT:

SUMMARY BOARD NOTIFICATION (83-140)

In accordance with the NRC procedures for Board Notifications, the enclosed are provided for your information. All 1982 and 1983 Board Notifications were reviewed for potential relevance to the licensing of Catawba. The following lists those which relate to the staff's licensing review of Catawba and have not been addressed in the staff's SER or FES.

BN	Number	Subject	Status and Method for Resolution
BN	82-01	Draft Report Entitled: "Interim Technical Assessment of the MARCH Code"	The staff considered this report in Catawba FES, Appendix E. The staff's assessment is that the uncertainties in the MARCH Code are much smaller than the uncertainties related to accidents as discussed in FES Section 5.9.4.5(7).
BN	82-03 82-13 82-40 82-87 82-111	Hydrogen Control for Ice Condenser Plants	Resolution will be handled under License Condition No. 10 in Catawba SER.
BN	82-04 82-10 82-18 82-87	Information with Regard to Pressurized Thermal Shock	Resolution will be handled under USI A-49.
BN	82 - 59 82 - 87	Information with Regard to Steam Generator Tube Integrity	Resolution will be handled under USI A-3.
BN	82-75	Accident Sequence Pre- cursor Report	Report was issued as background information and will be responded to through testimony, if necessary.
BN	82-81	Westinghouse Control Rod Drive Guide Tube Support Pin Failure	It appears that this issue does not apply to Catawba. Its resolution will be addressed in a supplement to the SER.

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BN	Number	Subject	Status and Method for Resolution
BN	82-93 82-107 82-124	Semiscale Test Results	Resolution will be handled under TMI Item II.K.3.30. Additional new information will result in a follow up BN.
BN	83-11	Preheater Type Steam Generator At Comanche Peak	Resolution is being handled on a generic basis (BN). For Catawba it will be handled under Open Item No. 8 of Catawba SER.
BN	83-13	EG&G Draft Report on Ranking of Systems	Potential letter may request information for staff review of this issue. It has not yet been decided if this issue will be handled on a generic basis (BN) or plant specific (SSER).
BN	83-17 83-44 83-57 83-105 83-126	USI-A-17 "Systems Inter- action in Nuclear Power Plants"	Potential future BN.
BN	83-26 83-38	Failure of Reactor Trip Breakers	Generic Letter 83-28 dated July 8, 1983, requests information for staff review of this issue. It has not yet been decided if this issue will be handled on a generic basis (BN) or plant specific (SSER).
BN	83-27	Additional RELAP-5 Cal- culation for Semiscale S-SR-2 Test	This information is related to BNs 82-93 and 82-107 regarding the feed and bleed cooling mode in PWRs. Its resolution will be addressed in a supplement to the SER.
BN	83-42	Unanalized Reactor Vessel Thermal Stress during Cooldown	It appears that this issue does not apply to Catawba. Its resolution will be addressed in a supplement to the SER.
BN	83-47 83-110	Need for Rapid Primary System Depressurization in PWRs and Status of PORV Issues for Westinghouse, B&W and CE Plants	Resolution will be handled under Open Item No. 9a in Catawba SER.

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BN 83-118

Board Notification Conto Nuclear Industry Companies by Ray Miller Inc. and Tube-Line Corporation

Resolution of this issue for Catawba cerning Materials Supplied will be handled in a supplement to the SER.

The staff will dispose of these issues as stated under "Status and Method for Resolution".

> Thomas M. Novak, Assistant Director for Licensing Division of Licensing

Enclosures: Board Notifications listed above

cc: See next page

-	NOTE: SEE P	REVIOUS WHITE	FOR CONCURREN	ICE	
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BN	Number	Subject	Status and Method for Resolution
BN	82-93 82-107 82-124	Semiscale Test Results	Resolution will be handled under TMI Item II.K.3.30. Additional new infor- mation will result in a follow up BN.
BN	83-11	Preheater Type Steam Generator At Conanche Peak	Resolution is being handled on a generic basis (BN). For Catawba it will be handled under Open Item No. 8 of Catawba SER.
BN	83-13	EG&G Draft Report on Ranking of Systems	NRC staff review of this report is being finalized. The final report will be issued in a follow up BN.
BN	83-17 83-44 83-57	USI-A-17 "Systems Inter- action in Nuclear Power Plants"	Recently completed BN (83-105) provides the final resolution of USI A-17. It is herewith provided in addition to BN 83-17, BN 83-44, and BN 83-57.
BN	83-26 83-38	Failure of Reactor Trip Breakers	Generic Letter 83-28 dated July 8, 1983, requests information for staff review of this issue. It has not yet been decided if this issue will be handled on a generic basis (BN) or plant specific (SSER).
BN	83-27	Additional RELAP-5 Cal- culation for Semiscale S-SR-2 Test	This information is related to BNs 82-93 and 82-107 regarding the feed and bleed cooling mode in PWRs. The staff position on feed and bleed is that it is not a design-basis requirement or necessary to meet Commission's regulations.
BN	83-47	Need for Rapid Primary System Depressurization in PWRs.	Resolution will be handled under Open Item No. 9a in Catawba SER.

The staff will dispose of these issues as stated under "Status and Method for Resolution".

Thomas M. Novak, Assistant Director for Licensing Division of Licensing

Enclosures: Board Notifications listed above

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Catawba Unit 1 Docket No. 50-413/414

Dr. Dixon Callihan
Dr. Richard F. Foster
Robert Guild, Esq.
Carole F. Kagan, Esq.
James L. Kelley, Esq.
J. Michael McGarry, III, Esq.
Palmetto Alliance
William L. Porter, Esq.
Mr. Jesse L. Riley
Richard P. Wilson, Esq.

Atomic Safety and Licensing Board Panel Atomic Safety and Licensing Appeal Panel Docketing and Service Section Document Management Branch

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Mr. Jeremiah J. Ray
Dr. Paul C. Shewmon
Dr. Chester P. Siess

Mr. David A. Ward

Mr. H. B. Tucker, Vice President Nuclear Production Dept. Duke Power Company 422 South Church Street Charlotte, North Carolina 28242

cc: William L. Porter, Esq.
Duke Power Company
P.O. Box 33189
Charlotte, North Carolina 28242

J. Michael McGarry, III, Esq. Debevoise & Liberman 1200 Seventeenth Street, N.W. Washington, D. C. 20036

North Carolina MPA-1 P.O. Box 95162 Raleigh, North Carolina 27625

Mr. F. J. Twogood Power Systems Division Westinghouse Electric Corp. P.O. Box 355 Pittsburgh, Pennsylvania 15230

Mr. J. C. Plunkett, Jr. NUS Corporation 2536 Countryside Boulevard Clearwater, Florida 33515

Mr. Jesse L. Riley, President Carolina Environmental Study Group 854 Henley Place Charlotte, North Carolina 28208

Richard P. Wilson, Esq. Assistant Attorney General S.C. Attorney General's Office P.O. Box 11549 Columbia, South Carolina 29211

Mr. Pierce K. Skinner Route 2, Box 179N York, South Carol ...a 29745 North Carolina Electric Membership Corp. 3333 North Boulevard P.O. Box 27306 Raleigh, North Carolina 27611

Saluda River Electric Cooperative, Inc. 207 Sherwood Drive Laurens, South Carolina 29360

Mr. Peter K. VanDoorn Route 2, Box 179N York, South Carolina 29745

James P. O'Reilly, Regional Administrator U.S. Nuclear Regulatory Commission, Region II 101 Marietta Street, Suite 3100 Atlanta, Georgia 30303

Robert Guild, Esq. P.O. Box 12097 Charleston, South Carolina 29412

Palmetto Alliance 2135 1/2 Devine Street Columbia, South Carolina 29205

Carol F. Kagan, Attorney Atomic Safety and Licensing Board Panel U.S. Nuclear Regulatory Commission Washington, D. C. 20555

SEP 1 2 1983

MEMORANDUM FOR:

Thomas Novak, Assistant Director for Licensing Darrell G. Eisenhut, Director

SUBJECT:

Division of Licensing SUMMARY BOARD NOTIFICATION FOR CATAMBA

I have determined that the information contained in your September 1, 1983 memo as modified and resubmitted on September 9, 1983 should be sent to the Board and parties for Catawba.

Issue this as BN 83-140.

Original signed by

Darrell G. Eisenhut, Director Division of Licensing

cc: K. Jabbour S. Black

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BN-83-140 *w/enclosure

September 13, 1983

Docket No(s)* 50-413/414
NRC PDR*
Local PDR*
PRC System*
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LB #4 r/f*
E. Adensam*
M. Duncan*
T. Novak
D. Eisenhut/R. Purple
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PPAS
J. Youngblood
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G. Knighton
R. Vollmer
H. Thompson
R. Mattson
S. Hanauer

Attorney, OELD

E. L. Jordan, DEQA; I&E
J. M. Taylor, DPR; I&E
W. J. Dircks, EDO (3)
E. Christenbury, OELD*
J. Scinto, OELD*
A. Bennette, OELD*

Project Manager K. Jabbour