Omaha Public Power District
P.O. Box 399 Hwy. 75 - North of Ft. Calhoun Fort Calhoun, NE 68023-0399
402/636-2000

February 15, 1995
LIC-95-0047

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Reference: Docket No. 50-285

SUBJECT: January 1995 Monthly Operating Report (MOR)

Enclosed please find the January 1995 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,

T. L. Patterson
Division Manager
Nuclear Operations

TLP/dll

Enclosures

c: LeBoeuf, Lamb, Greene & MacRae
L. J. Callan, NRC Regional Administrator, Region IV
S. D. Bloom, NRC Project Manager
R. P. Mullikin, NRC Senior Resident Inspector
R. T. Pearce, Combustion Engineering
R. J. Simon, Westinghouse
INPO Records Center

45.5129

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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

JANUARY 1995
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of January 1995, Fort Calhoun Station (FCS) operated at a nominal 100% power. Normal plant maintenance, surveillance, and equipment rotation activities occurred during the month, in addition to scheduled on line modification activities.

Vendor correspondence relative to a 10 CFR Part 21 notification indicated that the wire insulation material for PC-765-A/B/C/D (RPS Containment Pressure Switches) could be subject to breaking if the wires were moved excessively back and forth. The wiring from all switches was inspected. On January 17, PC-765-C was found to have cracked wiring insulation material on one lead. The Technical Specification 2.15 Limiting Condition for Operation (LCO) was entered so that the pressure switch could be replaced. The LCO was exited on January 18 after PC-765-C was restored to operability. All other switches were found to have no significant visible cracking; however, switch PC-765-D was replaced as a conservative measure since a spare switch was available. The remaining switches PC-765-A/B may be replaced in the future.

Three of four weekly new fuel shipments were received in preparation for the refueling outage; the final shipment was scheduled for February receipt.

On January 27, non-vital 120 VAC inverter #2 transferred to the bypass mode of operation. Technical Specification LCO 2.7.2 was entered with an 8-hour time limit. The inverter was inspected and no problems were found. It was then placed back into the normal mode of operation and declared operable. Later that day, the inverter again transferred to the bypass mode. The Technical Specification LCO was entered again and troubleshooting began. Because the corrective actions were not completed before the 8-hour time limit expired, a Notification of Unusual Event (NOUE) was declared, and preparations for a plant shutdown commenced at 1806 hours. At 1828 hours, the inverter was returned to service following replacement of two electronic cards and was declared operable. At 1848 the NOUE was terminated, and plant operation continued.

There were two NRC inspections completed during this reporting period:

IER 94-23 Monthly Resident Inspection

IER 95-01 Safety System Walkdown

The following LER was submitted during this reporting period:

LER No.	LER Date	Description
94-011	01/27/95	Failure to Satisfy Surveillance Requirement for Steam Generator Level Check.

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of January, no PORV or Primary System Safety Valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

RCS leak rate was steady throughout the month at a nominal .1 gpm with only minor fluctuations. No degrading trends were noted this month and the reactor coolant system continues to operate with minimal leakage. Except for variations observed during normal plant load changes and periodic increases from charging pump packing leaks, the leak rate has remained unchanged this cycle.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No.	Description
166	Changed the Technical Specifications to delete the surveillance requirements for Raw Water backup valves to the containment cooling coils and to delete the surveillance requirements in T.S. 3.2, Table 3-5, Item 6 for Raw Water interface valves.

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF JANUARY 1995

- Replaced containment purge exhaust pressure switches PC-765-C & PC-765-D
- Rebuilt component cooling water pump AC-3A
- Rebuilt corrosion inhibitor tank relief valve AC-36B
- Rebuilt charging pump CH-1A

- Inspected and replaced, as needed, gauges on regulators, piston rod boots and top/bottom snap rings for valve operators HCV-400A-0, HCV-401A-0, HCV-401B-0, HCV-401C-0 and HCV-401D-0

- Replaced defective breaker found during preventative maintenance activities on LPSI control valve HCV-331

- Replaced varistor on control element drive mechanism RC-10-02

- Straightened and replaced broken linkage on Diesel Generator #1 fresh air supply damper YCV-87[illegible]

- Replaced switch couplings on sequencer emergency standby-off selector switches 43-1/S1-1, 43-1/S1-2, 43-1/S2-1 and 43-1/S2-2

6. OPERATING DATA REPORT

   Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

   Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

   Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

   Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE FEBRUARY 07, 1995
COMPLETED BY M. L. EDWARDS
TELEPHONE 402-533-6929

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: JANUARY 1995

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	744.0	744.0	187178.0
12. Number of Hours Reactor was Critical	744.0	744.0	147161.9
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	744.0	744.0	145518.3
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1113178.0	1113178.0	193261878.5
17. Gross Elec. Energy Generated (MWH)..	378924.0	378924.0	63783806.2
18. Net Elec. Energy Generated (MWH)....	362181.4	362181.4	60853973.7
19. Unit Service Factor.....	100.0	100.0	77.7
20. Unit Availability Factor.....	100.0	100.0	77.7
21. Unit Capacity Factor (using MDC Net)	101.8	101.8	70.4
22. Unit Capacity Factor (using DER Net)	101.8	101.8	68.7
23. Unit Forced Outage Rate.....	.0	.0	4.0

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
REFUELING OUTAGE SCHEDULED TO COMMENCE ON MARCH 11, 1995, WITH PLANNED DURATION OF 49 DAYS.

25. If shut down at end of report period, estimated date of startup:

26. Units in test status (prior to comm. oper.):

		Forecast	Achieved
INITIAL CRITICALITY			
INITIAL ELECTRICITY	N/A		
COMMERCIAL OPERATION			

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	FEBRUARY 07,1995
COMPLETED BY	M. L. EDWARDS
TELEPHONE	402-533-6929

MONTH JANUARY 1995

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	487	17	486
2	487	18	486
3	487	19	486
4	487	20	487
5	487	21	487
6	487	22	488
7	486	23	487
8	487	24	487
9	487	25	487
10	487	26	487
11	487	27	487
12	487	28	487
13	487	29	486
14	487	30	486
15	487	31	486
16	486		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE February 7, 1995
COMPLETED BY T. C. Matthews
TELEPHONE (402) [illegible]-6938

REPORT MONTH January 1995

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Even: Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
None									

1
F: Forced
S. Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File(NUREG-0161)

5
Exhibit H - Same Source

(9/77)

Attachment IV
Refueling Information
Fort Calhoun Station Unit No. 1

Report for the month ending January 31, 1995

1. Scheduled date for next refueling shutdown.	March 11, 1995
2. Scheduled date for restart following refueling.	April 29, 1995
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment?	No
a. If answer is yes, what, in general, will these be?	N/A
b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.	No
c. If no such review has taken place, when is it scheduled?	Prior to April 1995
4. Scheduled date(s) for submitting proposed licensing action and support information.	No submittal planned
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	*
6. The number of fuel assemblies: a) in the core	133 Assemblies
b) in the spent fuel pool	570 Assemblies
c) spent fuel pool storage capacity	1083 Assemblies
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	2007 Outage

* OPPD is utilizing the CASMO-3/SIMULATE-3 codes for reactor physics related to analyses for Cycle 16. NRC approval for use of these codes/methods was received via a December 16, 1994 letter from S. D. Bloom (NRC) to T. L. Patterson (OPPD).

Prepared by [signature] Date 2-8-95