NRC FORM 366 APPROVED BY ONE U.S. NUCLEAR REGULATORY COMMISSION 180-0011 XPIRES 4-30-82 LICENSEE EVENT REPORT (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK SICIVICIS 1 0 0101 0 1 LICENSE NUMBER LICENSEE CODE CON'T REPORT L 0 5 0 0 3 9 5 0 0 8 3 1 8 3 0 9 1 3 8 3 9 SOURCE 40 41 DOCKET NUMBER 55 EVENT DATE 74 75 REPORT DATE 50 0 1 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10 On August 31, 1983, with the Plant in Mode 1, Pressurizer Power Operated Relief 0 2 Valve (PORV) PCV-445A was declared inoperable when a concern was identified over 0 3 the possible inadvertent opening of the valve upon a loss of power to Protection 0 4 Cabinet IV. The present plant design, with a loss of power to the protection 0 5 cabinet, would arm and actuate the PORV by means of the Cold Overpressure 0 6 Protection System. 0 7 0 8 80 COMP. VALVE CODE SUBCODE CAUSE COMPONENT CODE CODS Z 16 (12) X (15) (11 BI A (13) T I N IS TR CJ 0 9 19 5.1 13 CODE REPORT REVISION SEQUENTIAL NO REPORT NO LER/RO REPORT NUMBER 0 11 TI 10 813 11010 (17) 28 31 32 24 COMPONENT 26 NPRD-4 IME COMP. TTACHMENT FUTURE SHUTDOWN ACTION EFFEC 22 HOURS SUBMITTED SUB PLIER METHOD TAKEN ACTION ON PLANT Y 23 N 25 N 24 G ZIBF 10 WIL 12 20 Z 21 1010 (19) CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The cause is attributed to design error. The associated block valve for PORV 1 0 PCV-445A was closed and de-energized in accordance with Action Statement (a) of 1 1 Technical Specification 3.4.4 until the design error is corrected. 2 1 1 3 1 4 METHOD OF FACILIT (30) DISCOVERY DISCOVERY DESCRIPTION OTHER STATUS POWER STATUS Engineering Evaluation A (31) E 28 1 0 0 29 N/A 1 5 10 12 13 . ACTIVITY CONTENT LOCATION OF RELEASE (36) (35) AMOUNT OF ACTIVITY Z 33 Z 34 N/A N/A 1 6 80 10 11 PERSONNEL EXPOSURES NUMBER 0 0 0 37 Z 38 N/A 1 7 TE22 80 12 PERSONNEL INJURIES DESCRIPTION (41) NUMBER N/A 0 0 0 0 1 8 TYPE DESCRIPTION N/A 8309210353 830913 Z (42) 1 9 PDR ADOCK 05000395 PUBLICITY ISSUED DESCRIPTION (45) PDR NRC USE ONLY N a 14/13 2 0 (803) 345-5209 NE enne PHONE: NAME OF PREPARER C.V.J. McKinney

Mr. James P. O'Reilly LER No. 83-100 Page Two September 13, 1983

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

South Carolina Electric and Gas Company (SCE&G) has identified a concern with the channel assignment of an input to the Cold Overpressure Protection System (COPS). The concern involves the possible inadvertent opening of a Pressurizer Power Operated Relief Valve (PORV) due to a single failure.

During installation of the Reactor Vessel Level Instrumentation System (RVLIS) at Virgil C. Summer Nuclear Station, the vendor found it necessary to move a wide range hot leg temperature loop (T-433) from Channel I Protection Cabinet to Channel IV Protection Cabinet. This temperature loop is used by RVLIS for density compensation and was moved to a channel of the same separation group as the RVLIS channel to resolve a channel separation/independence concern. This present design creates a situation where a loss of power to Channel IV Protection Cabinet will inadvertently open PORV PCV-445A via the COPS.

CAUSE AND COPRECTIVE ACTIONS

The cause is attributed to design error as described above.

SCE&G and the vendor are evaluating the condition and a final report will be submitted upon the completion of the analysis detailing additional corrective action and a projected completion date. The associated block valve for PORV-445A has been closed and de-energized in accordance with Action Statement (a) of Technical Specification 3.4.4 until the design error is corrected. SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764 COLUMBIA, SOUTH CAROLINA 29218

O. W. DIXON, JR. VICE PRESIDENT NUCLEAR OPERATIONS 83 SEP19 P1:13 83 SEptember 13, 1983

Mr. James P. O'Reilly Regional Administrator U.S. Nuclear Regulatory Commission Region II, Suite 2900 101 Marietta Street, N.W. Atlanta, Ceorgia 30303

> SUBJECT: Virgil C. Summer Nuclear Station Docket No. 50/395 Operating License No. NPF-12 Fourteen Day Written Report LER 83-100

Dear Mr. O'Reilly:

Please find attached Licensee Event Report #83-100 for Virgil C. Summer Nuclear Station. This Fourteen Day Report is required by Technical Specification 6.9.1.12.(i) as a result of entry into Action Statement (a) of Technical Specification 3.4.4, "Relief Valves," on August 31, 1983.

Should there be any questions, please call us at your convenience.

Very truly yours,

O. W. Dixon, Jr.

CJM:OWD/mac/fjc Attachment

CC: V. C. Summer E. H. Craws, Jr. T. C. Nichols, Jr.,/O. W. Dixon, Jr. E. C. Roberts H. N. Cyrus Group/General Managers O. S. Bradham R. B. Clary C. A. Price A. R. Keon D. A. Lavigne J. F. Heilman C. L. Ligon (NSRC) G. J. Braddick J. C. Miller J. L. Skolds J. B. Knotts, Jr. I&E (Washington) Document Management Branch INPO Records Center NPCF File (Lic./Eng.)

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