NRC Form .00 (9.83) LICENSEE EVENT REPORT (LER)													U.S. MUCLEAR REGULATORY COMMISSION APPROVED OME NO. 3150-0104 EXPIRES 8/31/85 DOCKET NUMBER (2)							
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OM8 NO 3150-0104 EXPIRES 8/31/85

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On January 7, 1984, at 0030, with Unit 3 in Mode 3 at 0% power and a planned Reactor shutdown in progress, Control Element Assembly (CEA) 64 slipped thirty (30) inches. The Control Element Assembly Calculators (CEAC) detected the deviation of CEA 64 from its CEA group and generated a penalty factor. This penalty factor is used by the Core Protection Calculators (CPC's) in the Departure from Nucleate Boiling Ratio (DNBR) and Local Power Density (LPD) programs to account for local power peaks that could result from a misaligned CEA. The CPC's generated a DNBR and LPD trip on all four channels of the Reactor Protection System (RPS). All eight reactor trip breakers opened, fully inserting all CEA's not already inserted.

Since the secondary plant was shutdown and reactor power was at 0%, no operator action was required to stabilize plant parameters.

The Safety Analysis Report shows that a single CEA drop from 100% power, assuming the most reactive CEA and the worst case axial shape index, does not result in a violation of any specified acceptable fuel design limits. There are no credible circumstances under which this event would have resulted in the plant being outside design limits.

Since the unit was in Mode 3 and the RPS actuation was not required for plant safety, a four hour report pursuant to 10 CFR 50.72.b.2(ii) was not considered appropriate. Our continuing review of this matter has clarified that all RPS actuations, when the RPS function is not in bypass, should be reported pursuant to 10 CFR 50.72.b.2(ii). Consequently, operators have been instructed at preshift briefings to make the required report.

The cause of the misalignment of CEA 64 was sluggish operation of the CEA drive mechanism which causes the upper and lower grippers to sequence improperly. The event is similar to those reported in LER 83-062, 83-097, and 83-110 (Docket No. 50-362). As corrective action, the vendor has been requested to review this event and those previously reported and to provide appropriate corrective action.

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NRC Form 366A

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128 SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES STATION MANAGER

February 6, 1984

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Subject: Docket No. 50-362 30-Day Report Licensee Event Report No. 84-063 San Onofre Nuclear Generating Station, Unit 3

Pursuant to 10 CFR 50.73.a.2(iv), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the actuation of the Reactor Protection System.

If you require any additional information, please so advise.

Sincerely,

16. Laymes

J. G. HAYNES STATION MANAGER

RCClark:2454u:jll

Enclosure: LER No. 84-003

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, NRC Region V)

U. S. Nuclear Regulatory Commission Office of Inspection and Enforcement

Institute of Nuclear Power Operations (INPO)

