UPDATE REPORT — PREVIOUS REPORT DATE 11/13/83

NRC FORM 366 (12-81) 10 CFR 50 — U.S. NUCLEAR REGULATORY COMMISSION — LICENSEE EVENT REPORT — APPROVED BY OMB 3150-0011

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | ALBRF 2 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | (5)
LICENSEE CODE — LICENSE NUMBER — LICENSE TYPE — CAT

CON'T
0 1 | REPORT SOURCE L (6) | 0 5 0 0 0 2 6 0 (7) | 0 6 1 4 8 3 (8) | 0 9 1 4 8 3 (9)
DOCKET NUMBER — EVENT DATE — REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During normal operation following a rod adjustment and preconditioning, the LHGR
0 3 | and MAPLHGR limits were greater than the limits of Technical Specification 3.5.I
0 4 | and 3.5.J for one hour and 12-minutes. The maximum LHGR was noted to be 13.87
0 5 | and MAPLHGR was 12.00. Tech. Specs. 3.5.I and 3.5.J required both the LHGR and
0 6 | MAPHLGR values to be ≤13.4 and 11.838 respectively. There was no effect on
0 7 | the public health or safety. There are no redundant systems.
0 8 |

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	Z Z (11)	X (12)	Z (13)	Z Z Z Z Z Z (14)	Z (15)	Z (16)

(17) LER/RO REPORT NUMBER: EVENT YEAR		SEQUENTIAL REPORT NO.	OCCURRENCE CODE	REPORT TYPE	REVISION NO.
	8 3	0 3 3	0 3	X	1

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER (26)
X (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	Z (25)	Z 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | This event was caused by a xenon transient due to the load reduction and con-
1 1 | trol rod adjustment. Core flow was reduced to decrease load by 40 MWe and
1 2 | LHGR and MAPLHGR were brought to within limits. This is considered a random
1 3 | event and no recurrence control is required.
1 4 |

FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
E (28)	0 9 8 (29)	NA	A (31)	Engineer observation

ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)
0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES NUMBER	DESCRIPTION (41)
0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY TYPE (42)	DESCRIPTION (43)
Z	NA

IE22 1/1

PUBLICITY ISSUED (44) N — DESCRIPTION (45)

8309210155 830914
PDR ADOCK 05000260
S PDR

NRC USE ONLY

NAME OF PREPARER F. S. Poppell — PHONE (205) 729-0845

Tennessee Valley Authority
Browns Ferry Nuclear Plant

Form BF 17
BF 15.2
2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 83033R1 Technical Specification Involved 3.5.I & 3.5.J

Reported Under Technical Specification 6.7.2.b(2) * Date Due NRC 07/14/83

Event Narrative:

Unit 1 was in a refueling outage; unit 3 was operating at 100-percent power. These units were unaffected by this event. With unit 2 at 98-percent power following a rod adjustment and preconditioning, the nuclear engineer noted a maximum LHGR value of 13.87 and MAPLHGR value of 12.00, which were greater than the limits of 13.4 and 11.838 prescribed in Technical Specifications (TS) 3.5.I (MAPLHGR) and 3.5.J (LHGR.) There was no effect on the public health or safety. There are no redundant systems.

The cause of the event was a xenon transient due to the load reduction and control rod adjustment. Several OD-2 programs were run on the LPRM strings monitoring the limiting fuel bundles to refine the calculations. This reduced the LHGR and MAPLHGR, but the new values were still above 13.4 and 11.838, respectively. Core flow was then reduced, decreasing the load by 40 MWe and bringing LHGR and MAPLHGR back to within technical specification limits. LHGR and MAPLHGR were noted out of limits at 1913 hours and were restored to within limits at 2025 hours (one hour 12-minutes.) This is considered a random event and no further recurrence control is required.

* Previous Similar Events:

BFRO-50-259/80053, 80056, 80078
260/81005, 81006, 81007
296/79003, 81018

Retention: Period - Lifetime: Responsibility - Document Control Supervisor

*Revision: JRP

TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401
1750 Chestnut Street Tower II

83 SEP 19 A9:55

September 14, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE REPORT BFRO-50-260/83033 - REVISION 1

The enclosed report is a supplement to my letter to you dated July 13, 1983 concerning core maximum fraction of limited power density and core maximum average power ratio which exceeded values prescribed in technical specification requirements caused by xenon transient because of a load reduction and control rod adjustment. This report is submitted in accordance with Browns Ferry unit 2 Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

Herbert Abercrombie

H. J. Green
Director of Nuclear Power

Enclosure
cc (Enclosure):
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U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

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Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

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