

LICENSEE EVENT REPORT

Update Report
Previous Report Date: 9-6-79

CONTROL BLOCK: _____ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | N C B E P I | 2 | 0 0 - 0 0 0 0 0 0 - 0 0 | 3 | 4 1 1 1 1 | 4 | _____ | 5

CON'T
0 1 | REPORT SOURCE | L | 6 | 0 5 0 - 0 3 2 5 | 7 | 0 8 0 8 7 9 | 8 | 0 1 3 1 8 4 | 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | While performing PT-01.1.13P, High Steam Line Radiation Channel Alignment and
0 3 | Functional Test, Main Steam Line Hi Rad Monitor B was found to be actuating at
0 4 | 3.95 X background while the allowable limit is ≤ 3.5 X background. This event did not
0 5 | affect the health and safety of the public.
0 6 | _____
0 7 | _____
0 8 | _____ Technical Specifications 2.2.1, 6.9.19a _____

0 9 | SYSTEM CODE | M C | 11 | CAUSE CODE | E | 12 | CAUSE SUBCODE | E | 13 | COMPONENT CODE | I N S T R U | 14 | COMP. SUBCODE | Y | 15 | VALVE SUBCODE | Z | 16

17 | LER/RO REPORT NUMBER | 7 9 | EVENT YEAR | 0 5 6 | SEQUENTIAL REPORT NO. | 0 3 | OCCURRENCE CODE | L | REPORT TYPE | 1 | REVISION NO. | 1 |

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The cause of the out-of-tolerance reading was attributed to instrument drift. The
1 1 | monitor was recalibrated as per PT-01.1.12PC and left operating satisfactorily. No
1 2 | further action is planned regarding this event.
1 3 | _____
1 4 | _____

1 5 | FACILITY STATUS | E | 28 | % POWER | 0 9 5 | 29 | OTHER STATUS | NA | 30 | METHOD OF DISCOVERY | B | 31 | DISCOVERY DESCRIPTION | Periodic Test | 32

1 6 | ACTIVITY RELEASED OF RELEASE | Z | 33 | Z | 34 | AMOUNT OF ACTIVITY | NA | 35 | LOCATION OF RELEASE | NA | 36

1 7 | PERSONNEL EXPOSURES NUMBER | 0 0 0 | 37 | TYPE | Z | 38 | DESCRIPTION | NA | 39

1 8 | PERSONNEL INJURIES NUMBER | 0 0 0 | 40 | DESCRIPTION | NA | 41

1 9 | LOSS OF OR DAMAGE TO FACILITY TYPE | Z | 42 | DESCRIPTION | NA | 43

2 0 | PUBLICITY ISSUED | N | 44 | DESCRIPTION | _____ | 45

8402140005 840131
PDR ADOCK 05000320
S PDR

NRC USE ONLY

NAME OF PREPARER: Mary T. Allen

PHONE: (919) 457-9521

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Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429
January 31, 1984

FILE: B09-13510C
SERIAL: BSEP/84-0207

Mr. James P. O'Reilly, Administrator
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street N.W.
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
SUPPLEMENT TO LICENSEE EVENT REPORT 1-79-056

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9a of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 1, the enclosed supplemental Licensee Event Report is submitted. The original report fulfilled the requirement for a written report within thirty (30) days of a reportable occurrence and both are in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MTA/mcg/LETCG2

Enclosure

cc: Mr. R. C. DeYoung
NRC Document Control Desk

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