

## LICENSEE EVENT REPORT

CONTROL BLOCK: 1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	N	Y	J	A	F	1	2	0	0	-	0	0	0	0	-	0	0	0	3	4	1	1	1	1	4		5									
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35									
LICENSEE CODE														LICENSE NUMBER										LICENSE TYPE										CAT		58	

CON'T

0	1	L	6	0	5	0	0	0	3	3	3	7	0	8	1	9	8	3	8	0	9	0	2	8	3	9					
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35			
REPORT SOURCE		DOCKET NUMBER										EVENT DATE										REPORT DATE									

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During a special inspection of the main steam line pipe supports and constraints out-

0 3 | board of the containment MSIV's, it was noted that the support frame above trunnion

0 4 | H29-351 was bent. This event is being reported under TS 6.9A4.2d. The damage was

0 5 | minor and resulted in additional clearance which would have allowed greater pipe

0 6 | movement. The occurrence is not a condition that could have caused a threat to the

0 7 | health and safety of the general public.

0	9	H	B	11	B	12	B	13	P	I	P	E	X	X	14	E	15		16	8	3		0	2	6	0	3	L		0	
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	
LER/RO REPORT NUMBER		EVENT YEAR		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE								COMP. SUBCODE		VALVE SUBCODE		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.					
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS								ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER									
F		Z		Z		Z		00000										Y		A		Z9999									
33		34		35		36		37								40		41		42		43		44							

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | This trunnion and other similar trunnion and associated support steel were reanalyzed

1 1 | by the plants architect engineer. Results are reported in an enclosure. To avoid

1 2 | a costly and time consuming reanalysis of the support embedment, two additional

1 3 | supports were installed to carry the loads previously carried by the wall embedments.

1 4 | CONTINUED ON ATTACHED SHEET.

1	5	H	28	0	0	0	29		C	31	observation by plant personnel
7	8	9	10	11	12	13	14	15	16	17	18
FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION			
ACTIVITY CONTENT RELEASED OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE							
1	6	Z	33	Z	34	NA					
7	8	9	10	11	12	13	14	15	16	17	18
PERSONNEL EXPOSURES		NUMBER		TYPE		DESCRIPTION					
1	7	0	0	0	37	Z	38	NA			
7	8	9	10	11	12	13	14	15	16	17	18
PERSONNEL INJURIES		NUMBER		DESCRIPTION							
1	8	0	0	0	40		NA				
7	8	9	10	11	12	13	14	15	16	17	18
LOSS OF OR DAMAGE TO FACILITY		TYPE		DESCRIPTION							
1	9	Z	42		NA						
7	8	9	10	11	12	13	14	15	16	17	18
PUBLICITY		ISSUED		DESCRIPTION							
2	0	N	44		NA						
7	8	9	10	11	12	13	14	15	16	17	18

NAME OF PREPARER ROBERT BAKER PHONE:

8309190204 830902  
PDR ADDCK 05000333  
S PDR

IE2211

NRC USE ONLY

POWER AUTHORITY OF THE STATE OF NEW YORK  
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER - 83-026

PAGE 2 of 2

The plants architect engineer investigated the situation and reanalyzed this specific trunnion and similar trunnions on other steam lines and associated supporting steel. This analysis indicated the following:

- a) The original design clearance between the trunnion and supporting steel was insufficient to allow for thermal growth. Recent visual inspection indicates that the currently existing clearance is satisfactory.
- b) The minor bending in the flange of the supporting beam appears to have resulted because the beam was installed slightly rotated. This condition had originally been evaluated as satisfactory as the result of an observation made in 1979.

The supports and structural steel excluding the embedments were fully reanalyzed and found to meet all code requirements. To save time and expense, it was decided to forego the time consuming and costly reanalysis of embedments and to install two additional structural members to carry the loads previously carried by the wall embedments. This installation was completed on September 1, 1983.

EOP

Copy to:

J.P. Bayne - NYPA  
R. Burns - NYPA  
L. Guaquil - NYPA  
C. McNeill - NYPA Site  
V. Walz - NYPA Site

E. Siskin  
P. Dunlop  
H. Faery  
D. Cirrone  
K. Chu  
Central Job Book  
Task 08 Job Book (08325)

TELECOPIED

TO: C. McNeill

FROM: D. J. Cirrone

DATE: 9/1/83

PAGE: 1 of 2

TIME: 3:45

Mr. J. P. Bayne  
Executive Vice President  
New York Power Authority  
123 Main Street  
White Plains, NY 10601  
Attn: Mr. L. Guaquil

August 31, 1983

J.O. No. 12966.08  
PAS-26008

MAIN STEAM LINE SUPPORTS

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

NYPA letter JPS-83-29, dated July 6, 1983, forwarded a letter from Target Technology, Ltd. (TTL) dated June 30, 1983 which raises a number of questions concerning the adequacy of piping and pipe support analyses for J.A. FitzPatrick Nuclear Power Plant. Stone & Webster's reply to NYPA's letter will be forwarded by separate correspondence.

One specific question which has arisen as a result of NYPA's and Stone & Webster's evaluation of the questions raised by TTL concerns a minor deformation noted in the support frame above trunnion H29-351. Stone & Webster's analyses of the specific trunnion, similar trunnions on other main steam lines and the associated supporting steel have indicated the following:

- a) The upper gap between the trunnion and the supporting steel as shown in the original Bergen-Paterson drawing was insufficient to allow for thermal growth. To permit satisfactory thermal expansion the sum of the upper and lower gaps should be at least 1/4 inch. The gaps that exist today, as confirmed by a recent inspection, are technically satisfactory.
- b) The minor bending in the flange of the support beam appears to have resulted because that beam was installed slightly rotated. Therefore, any upward force due to thermal growth or turbine trip loads would be sufficient to bend the outer edge of the flange as was observed. As was reviewed with NYPA and TTL in July 1979, this minor distortion does not degrade the capability of the structural support to function properly.

TELECOPIED  
TO: C. A. McNeill  
From: D. J. Cirrone  
3:50pm on 9-1-83

COPY

JPB

-2-

August 31, 1983

- c) The supports and structural steel excluding the embedments were fully reanalyzed and found to meet all code requirements. To save time and expense, it was decided to forego the time consuming costly reanalyses of embedments and to recommend that two additional structural members be added to carry the loads previously carried by the wall embedments. The suggested design modification to accomplish this was forwarded to NYPA with SWEC letter PAS-26001, dated August 17, 1983. Please note that NYPA should require as-built verification after installation of the support modifications. The supporting calculations are lengthy and will be maintained on file at Stone & Webster. Copies will be forwarded upon request.

If you have any questions concerning the above, please contact us.

*D. J. Cirrone*

D. J. Cirrone  
Project Engineer

BJS/KYC:pcw

TELECOPIED

TO: C. McNeill

FROM: D. J. Cirrone

DATE: 9/1/83

PAGE: 2 of 2

TIME: 3:05 p.m.



JAFP 83-0921  
September 7, 1983

Dr. Thomas E. Murley  
Regional Administrator  
United States Nuclear Regulatory Commission  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

SUBJECT: DOCKET NO. 50-333  
LICENSEE EVENT REPORT: 83-026

Dear Dr. Murley:

We have enclosed the subject Licensee Event Report in accordance with Section 6.0 of Technical Specifications and USNRC Regulatory Guide 1.16.

If there are any questions concerning this report, please contact Mr. Robert Baker at (315) 342-3840, extension 261.

Very truly yours,

A handwritten signature in cursive script, appearing to read 'Corbin A. McNeill, Jr.', followed by a horizontal line.

CORBIN A. McNEILL, JR.  
RESIDENT MANAGER

CAM:RB:ls  
ENCLOSURE

CC; USNRC Document Control Desk (1)  
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Internal Power Authority Distribution  
NRC Resident Inspector  
Document Control Center  
LER/OR File

TE27  
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