LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

AUGUST 1983

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373 LICENSE NO. NPF-11

IE24

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I. INTRODUCTION

The LaSalle Nuclear Power Station Unit One is a Boiling Water Reactor with a designed electrical output of 1078 MWe net, located in Marseilles, Illinois. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. The plant is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit has not commenced commercial generation of power.

This report was complied by Diane L. Lin, telephone number (815)357-6761, extension 499.

II. SUMMARY OF UNIT OPERATING EXPERIENCE FOR UNIT ONE

August 1-12 The unit started the reporting period at approximately 54% reactor power. At 0221 hours on August 4 reactor power was reduced to restart the RR pumps. Reactor power was 36%. At 0700 hours on August 4 reactor power was increased to 63%. At 0445 hours on August 7 reactor power was 83%. At 0200 hours on August 8 the reactor was at 100% power. On August 9 at 0250 hours reactor power was reduced from 95% to 64%. At 2212 on August 9 reactor power was 92%. On August 11 at 1728 hours reactor power was decreased to approximately 70% for reactor recirc testing. At 0945 hours on August 12 reactor power was increased to 90% via reactor recirc system. On August 12 at 1250 hours the reactor scrammed while performing LIS-NR-04 the section on APRM Channel C, flow bias reference set point. Received a downscale trip which caused the RR FCV to open and the unit to scram. The reactor was critical for 276 hours and 50 minutes.

August 13-31 The reactor went critical at 0600 hours on August 13. At 1413 hours on August 13 the main generator was synchronized to the grid and loaded. At 2300 hours on August 13 reactor power was 47%. At 1500 hours on August 14 reactor power was 71%. On August 16 at 2100 hours reactor power was 95%. At 1705 hours on August 18 reactor power was reduced due to a GSEP alert initiated because of the 'A' VE Charcoal bed getting wet. At 2323 hours on August 18 reactor power was 52%. At 1700 hours on August 19 reactor power was 80%. On August 20 at 0932 the reactor scrammed due to low feedwater flow. On August 22 this outage was continued to replace the "B" recirc pump seals, heater drain system modifications and local leak rate testing. Also the main control valves were converted from partial arc to full arc steam admission. The reactor was critical for 171 hours and 32 minutes.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE

- A. Amendments to Facility License or Technical Specifications.

 There were no amendments to the facility license or technical specifications during the reporting period.
- B. Facility or Procedure Changes Requiring NRC Approval.
 There were no facility or procedure changes requiring NRC approval during the reporting period.
- C. Tests and Experiments Requiring NRC Approval.
 There were no tests or experiments requiring NRC approval during the reporting period.
- D. Corrective Maintenance of Safety Related Equipment.

 The following tables present a summary of safety-related maintenance completed on Unit One during the reported period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Component Name, Cause of Malfunctions, Results and Effects on Safe Operation, and Corrective Action.

WORK REQUEST	LER .			CAUSE OF MALFUNCTION RESULTS AND EFFECTS ON SAFE OPERATION	
L23265		D/G Air Receiver Pressure Indicator Isolation Valve	Bad Valve	Valve leaks	Replaced valve
L23828		Battery Unit One 250 V Charger	Bad breaker	Breaker is weak	Replaced <u>breaker</u>
L25831	83-080/03L-0	LPCS Water Leg Pump	Locked bearing	Pump frozen with the outboard bearing locked.	Replaced bearings, oil seals, mechanical seal, shaft, shaft sleeve
L25902	83-082/03L-0	Rx Bldg. Vent Exhaust Plenum Rad Monitor	Bad GM tube	Spurious upscale trip Observed.	Replaced GM tube
L26033 ·	83-077/03L-0	Ammonia Dectecor	Bad master fault bulb	Optical unit not working	Replaced master fault bulb
L26044	83-083/03L-0	SBGT WRGM	Blown fuse	Pump didn't start in i	Replaced fuse
L26101	83-078/03L-0	LPCS Injection Valve Low Pressure Switch	Switch out-of-adjustment	Alarm is up when pressure is not low	Recalibrated switch
L26118 L26119		Unit 1 Fuel Pool Vent PRM A & B	Bad detector tube	Background response increasing over time	Replaced detector tube
L26207		RHR Service Water Pump A-B Cubicle Temperature Hi Alarm	Bad fuse	Cubicle has high Temperature	Replaced fuse
L26576	,	HPCS Discharge Pressure Gauge	Air trapped on high side of gauge	Gauge reads lower than local gauge	Vented air
L26617		Division I Post Loca Hydrogen Monitor	Motor drive out-of alignment. Pen drive & spring loose	Recorder reads ~ 2% Hydrogen with the unit S/D	Realigned motor drive. Tightened pen drive and spring

WORK REQUEST LER COMPONENT CAU		CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION	
L26654		OPL58JA Pen #1	Problem due to heat	Pen is stuck downscale	Fixed cooling fans
Ľ26657 L26658		Chlorine Detecter A & B	Bad Felt wick	Electronic drip rate exceeds 4 minutes as specified in vendor manua	Replaced felt wick
L26707		Leak Detector△T Recorder	Bad amplifier	Recorder pegs high and/or low when it is pulled out and pushed in	Replaced amplifier
L26878		Standby Gas Treatment system Effluent and lodine Recorder	Bad chart drive motor	Paper does not advance	Replaced chart drive motor
L27036		"1B" LPCI Testable Check	Appears Spring tension was set too tight	Valve binds requiring wrench assist to open and close	Cleaned and relubricated bearing. Reset timing. Adjusted spring tension.
L27037		"1C" LPCI Testable Check	Limit Switch dirty	Valve did not cycle 'properly'	Cleaned limit switch
L27181		250 VDC Battery Charger	Blown fuse	Keeps tripping both AC feeds and DC output breakers	Replaced fuse

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, August 1 through August 31, 1983. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

Licensee Event Report Number	Date	Title of Occurrence
83-076/03L-0	7-09-83	RCIC Steam Line High Flow Isolation
83-077/03L-0	7-16-83	B System Ammonia Detector
83-078/03L-0	7-19-83	Reactor Vessel Low Pressure LPCS Injection Valve Permissive
83-079/03L-0	7-08-83	Reactor Low Pressure RCIC Isolation
83-080/03L-0	7-07-83	LPCS Water Leg Pump
83-081/03L-0	7-18-83	Hole in FW Pipe
83-082/03L-0	7-11-83	Stack WRGM
83-083/03L-0	7-17-83	SBGT WRGM
83-084/03L-0	7-18-83	PCIS Valve 1CM027 Failed to Isolate
83-085/03L-0	7-22-83	Failure of a RHR SW PRM Sample Pump
83-086/03L-0	7-24-83	Unit 2 Division 1 Battery Charger
83-087/03L-0	7-18-83	Bypass Valves Failed Open/ECCS Initiation
83-088/03L-0	7-27-83	Illinois Dept. of Nuclear Safety Uncontrolled Release
83-089/03L-0	7-25-83	Failure of Unit 2 SBGT Heaters to Energize
83-090/03L-0	7-26-83	Leak in Line Designed to Contain Radioactive Liquid

Licensee Event Report Number	Date	Title of Occurrence
83-091/03L-0	8-04-83	Reactor Vessel Lo Lo Water Level
83-092/03L-0	8-01-83	Control Rod 34-43 Accumulator Inop
83-093/03L-0	8-01-83	Control Rod 34-47 Full In Light Inop
83-094/03L-0	8-05-83	Radwaste Discharge Sample Pump Seal Failure

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

VI. UNIQUE REPORTING REQUIREMENTS

- A. Main Steam Relief Valve Operations for Unit 1

 There were no main steam relief valve operations for the reporting period.
- B. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period:

Outage No.	Equipment	Purpose of Outage		
1-777-83	HPCS Water Leg Pump	Repair Oil Leak		
1-796-83	HPCS Water Leg Pump	Replace Pump		

C. Off-Site Dose Calculation Manual

There were no changes to the Off-Site Dose Calculations
Manual during this reporting period.

- D. Radioactive Waste Treatment System
 There were no changes to the Radioactive Waste Treatment
 System during this reporting period.
- E. Process Control Program

There were no changes to the Process Control Program during this reporting period.

LTP-300-7 Revision 2 November 13, 1979

OPERATING DATA REPORT

DOCKET NO.	050-373
UNIT	LaSalle One
DATE	Sept. 7, 1983
COMPLETED BY	Diane L. Lin
TELEPHONE	(815) 357-6761

OPERATING STATUS			
1. REPORTING PERIOD: August 1983 GROSS HOURS IN			
2. CURRENTLY AUTHORIZED POWER LEVEL (MWI): 100% MAX. DESIGN ELECTRICAL RATING (MW+N+1): 1078	DEPEND, CAPACI	TY (MW+Nes):	0
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MW-Net):			
4. REASONS FOR RESTRICTION (IF ANY):			
	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	448.4	1029.0	6457.6
6. REACTOR RESERVE SHUTDOWN HOURS	0	0	0
7. HOURS GENERATOR ON LINE	440.1	2557.98	4415.68
& UNIT RESERVE SHUTDOWN HOURS	0	- 0,	0
9. GROSS THERMAL ENERGY GENERATED (MWH)	1131742	4567722	6708301
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	361365	1383413	1743503
11. NET ELECTRICAL ENERGY GENERATED (MWH)	342556	1268919	1729694
12 REACTOR SERVICE FACTOR	<u>NA</u>	NA	NA
13. REACTOR AVAILABILITY FACTOR	NA	· NA	NA
14 UNIT SERVICE FACTOR	· NA	NA_	NA
15. UNIT AVAILABILITY FACTOR	NA	NA	NA
16. UNIT CAPACITY FACTOR (Using MDC)	. NA	NA	NA
17. UNIT CAPACITY FACTOR (Using Design MWe)	NA NA	NA	NA
18. UNIT FORCED OUTAGE RATE	NA NA	NA	NA
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS ITYPE, DATE, A	NO DURATION O	F EACH):	
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF	STARTUP:		
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):	FORECAST	ACHIEVED	
INITIAL CRITICALITY		6/21/82	
INITIAL ELECTRICITY		9/4/82	
COMMERCIAL OPERATION	9/23/83		

ATTACHMENT D

LTP-300-7 Revision 2 November 13, 1979

OPERATING DATA REPORT

	DOCKET NO.	050-373	- 17
	UNIT	LaSalle 0	ne
		August 8.	THE PARTY OF THE PARTY
	COMPLETED BY		
	TELEPHONE	(815) 357-	6701
PERATING STATUS			
REPORTING PERIOD: July 1983 GROSS HOURS	N REPORTING PER	1100: 744	
CURRENTLY AUTHORIZED POWER LEVEL (MWt): 100% MAX DESIGN ELECTRICAL RATING (MW+N+t): 1078	DEPEND. CAPACI	TY (MWe-Net): _	0
L POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MW-Net):	1/A		
REASONS FOR RESTRICTION (IF ANY):			
	THIS MONTH	YR TO DATE	CUMULATIVE
S. NUMBER OF HOURS REACTOR WAS CRITICAL	580.6	2371.8	6009.2
S. REACTOR RESERVE SHUTDOWN HOURS	0	0	2075 50
T. HOURS GENERATOR ON LINE	501.5	2117.88	3975.58
& UNIT RESERVE SHUTDOWN HOURS	0		
9. GROSS THERMAL ENERGY GENERATED (MWH)	1028597	3435980	5576559
O. GROSS ELECTRICAL ENERGY GENERATED (MWH)	321018	1022948	1543347
1. NET ELECTRICAL ENERGY GENERATED (MWH)	300122	926363	1387138
2. REACTOR SERVICE FACTOR	NA	_NA	NA
3. REACTOR AVAILABILITY FACTOR	<u>NA</u>	NA NA	NA
A UNIT SERVICE FACTOR	<u>NA</u>	NA	. NA
S. UNIT AVAILABILITY FACTOR		NA NA	NA
16. UNIT CAPACITY FACTOR (Using MDC)	<u>NA</u>	NA NA	NA NA
17. UNIT CAPACITY FACTOR (Using Design MWe)		NA	NA NA
IR UNIT FORCED OUTAGE RATE	<u>NA</u>	NA	<u>NA</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS ITYPE, DATE,	AND DURATION OF	F EACH):	
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE O	F STARTUP:		
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):	FORECAST	ACHIEVED	
		6/21/83	
INITIAL CRITICALITY	A 4 4 4 4 1 1 1	9/4/82	
INITIAL ELECTRICITY	0/22/92		
COUNTED CO. CORRESTION	9/23/83		

ATTACHMENT A AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	050-373
UNIT	LaSalle One
DATE	Sept: 7, 1983.
COMPLETED BY	Diane L. Lin
TELEPHONE	(815) 357-6761

			TELEPHONE	(815) 39	57-6761
HONTH	August 1983				
DAY	AVERAGE DAILY POWER LEVEL (HWE-Net)	. DAY		DAILY P	OWER LEVEL
1	632	17.		987	
2.	646	18.		848	
3.	516	19.		846	***
4.	599	20.		377	
5.	683	21.		0	
6.	771	22.		0	100
7.	934	23.		0	
8.	1014	24.		0	5-89
9.	867	25.		0	
10.	990	26.		. 0	. 20
11.	941	27.		0	
12.	493	28.		0	- 4
13.	101	29.		0	440
14.	513	30.		0	
15.	743	31.		0	
16.	911				

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT HONTH August 1983

DOCKET NO. 050-373

UNIT NAME

LaSalle One

DATE

Sept. 7, 1983

COMPLETED BY

Diane L. Lin

TELEPHONE

(815) 357-6761

		TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD O SHUTTING DO THE REACTOR REDUCING PO	WN OR	CORRECTIVE ACTIONS/COMMENTS
NO.	DATE	2: 2CHEDOLED			5		Reduced power from 95% to 64%
30	8/09/83	F	0.	Н.			
31	8/11/83	S	0	. Н	5		Reduced power for Reactor Recirc. testing
32	8/12/83	F	17.2	В	3 .		Reactor scram due to APRM Hi Hi Alarm
					5		'A' VE Charcoal Bed got wet
33	8/18/83	F *	. 0	Α			
34	8/20/83	F	278.5	A .	3.		Reactor scram due to low Reactor water inlet flow

September 7, 1983

Director, Office of Management Information and Program Control United States Nuclear Regulatory Commission Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station, Unit One, for the period covering August 1 through August 31, 1983.

Very truly yours,

C & Sargent

G. J. Diederich

Station Superintendent

GJD/DLL/rmp

Enclosure

xc: J. G. Keppler NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright III. Dept. of Nuclear Safety
D. P. Galle CECo
D. L. Farrar CECo
INPO Records Center

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