

LER No. 83-007
Omaha Public Power District
Fort Calhoun Station Unit No. 1
Docket No. 05000285

ATTACHMENT NO. 1

Safety Analysis

The Reactor Protective System (RPS) is so designed that no single failure can prevent the safe and systematic shutdown of the reactor if required.

During the time the RPS "B" channel for Axial Power Distribution (APD) was inoperable, the remaining three redundant RPS Axial Power Distribution Channels were operable, available and fully capable of performing their design function, i.e., providing adequate reactor protection.

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Corrective Action

The positive Axial Power Distribution setpoint was found to be out of specification on July 22, 1983. Maintenance Order No. 21116 was initiated to investigate and repair the problem. Consequently, it was discovered that a Bell and Howell dual potentiometer module (model #384612-01) had failed and caused the positive APD setpoint to drift. The "B" channel of the RPS for Axial Power Distribution was placed in bypass until the failed dual potentiometer module was replaced with an equivalent module per Engineering Evaluation and Assistance Request (EEAR) FC-83-97 and Safety Related Design Change Order (SRDCO) FC-83-46 on July 29, 1983. The new module was a Bell and Howell model #20-320 dual potentiometer module. The new module was calibrated per applicable sections of CP-B/APD and satisfactorily tested per applicable sections of surveillance test ST-RPS-12, F.2 prior to being returned to service.

The Reactor Protective System will continue to be monitored by both visual inspection and surveillance testing in an effort to detect future failures. In addition, an Engineering Evaluation and Assistance Request (EEAR) has been initiated (EEAR FC-83-93) to investigate whether or not recent dual potentiometer module failures are generic and if so to investigate a reasonable course of action.

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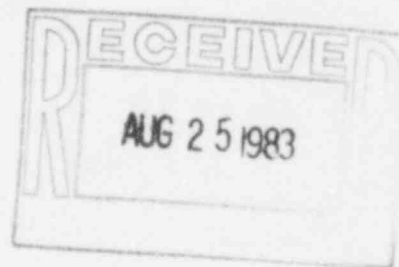
ATTACHMENT NO. 3

Failure Data

This is the fourth failure of a dual potentiometer module in the Reactor Protective System. The previous failures were reported per LER's 79-002, 83-005 and 83-006.

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102
402/536-4000

August 22, 1983
FC-600-83



Mr. W. C. Seidle, Chief
Reactor Project Branch 2
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

Subject: Fort Calhoun Station Unit No. 1
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Dear Mr. Seidle:

In accordance with the Fort Calhoun Station's Technical Specifications, Omaha Public Power District, as holder of Facility Operating License DPR-40, submits three copies of Licensee Event Report 83-007 (regarding Technical Specification 5.9.2.b.1) to satisfy requirements of Regulatory Guide 1.16.

Sincerely,

A handwritten signature in dark ink, appearing to read "W. C. Jones".

W. C. Jones
Division Manager
Production Operations

WCJ/GRP:jmm

Enclosures

cc: Director, Office of Management
Information & Program Control (3)
Director, Office of Inspection
& Enforcement (30)
Institute of Nuclear Power Operations

SARC Chairman
PRC Chairman
Fort Calhoun File (2)
Mr. L. A. Yandell, NRC Senior Resident
Inspector