

COMANCHE PEAK STEAM ELECTRIC STATION

TRAINING MANUAL

CONTROLLED COPY NO. 058

**FOR INFORMATION
ONLY**

LICENSED OPERATOR REQUALIFICATION TRAINING PROGRAM

PROCEDURE NO. TRA-204

REVISION NO. 3

SAFETY-RELATED

SUBMITTED BY:

Joe Brackney for RR Wistrand
ADMINISTRATIVE SUPERINTENDENT

DATE: 8-10-83

APPROVED BY:

R. L. Jones
MANAGER, PLANT OPERATIONS

DATE: 8/11/83

8308300629 830823
PDR ADOCK 05000445
V PDR

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1.0 Purpose

The purpose of the licensed operator requalification program is to ensure a proficient operating staff. This program is established to comply with the requirements of 10 CFR 55, Appendix A, while providing as individualized a retraining program as possible.

2.0 Applicability

This procedure applies to all USNRC licensed personnel. This procedure becomes effective upon issuance of USNRC Licenses to station operating personnel.

3.0 Definitions

3.1 Reactor Operator (RO) - any individual who possesses an operator's license pursuant to Title 10, Code of Federal Regulations, Part 55, "Operator's Licenses".

3.2 Senior Reactor Operator (SRO) - any individual who possesses a senior operator's license pursuant to Title 10, Code of Federal Regulations, Part 55, "Operator's License".

4.0 Instructions

4.1 Responsibilities

4.1.1 The Manager, Plant Operations has the overall responsibility for ensuring that the CPSES staff is adequately trained and fully qualified to operate equipment/systems at CPSES in accordance with regulatory and station guidelines.

4.1.2 The Director, Nuclear Training provides any requested training services to the Operations Department, and is responsible for documenting all training administered to that group.

4.1.3 The Operations Superintendent is responsible to the Manager, Plant Operations for identifying training needs, and developing and implementing a training program for the Operations Department.

4.1.4 The Operations Supervisor is responsible to the Operations Superintendent for the following training functions:

4.1.4.1 Recommending and supervising the development of course curricula.

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4.1.4.2 Reviewing training records, lesson material, and qualifications.

4.1.4.3 Evaluating the Requalification Program effectiveness, and recommending changes as necessary.

4.1.4.4 Scheduling requalification training.

4.1.5 The Shift Supervisor(s) are responsible to the Operations Supervisor for the following:

4.1.5.1 Identifying the training needs of the Reactor Operators.

4.1.5.2 Identifying and recommending any specialty training needed by the Reactor Operators to adequately operate equipment/systems at CPSES.

4.1.5.3 Observing Reactor Operator performance of assigned tasks and determining if specific skills have been obtained and are being maintained.

4.1.5.4 Notifying the Operations Supervisor of inability to complete requalification program activities as scheduled.

4.2 Lectures

The Licensed Operator Requalification Program shall include pre-planned training sessions conducted on a regular and continuing basis. The training sessions will include two types of lecture series as follows:

4.2.1 Fundamental Review Lecture Series

The Fundamental Review training sessions cover areas in which the knowledge required of a licensed individual is relatively constant.

4.2.1.1 Lecture Series Topics

The Fundamental Review lecture topics are selected on an as-needed basis and include the following:

1. Principles and Theory of Nuclear Power Plant operation.

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2. Fundamentals and Theory of Fluid Flow, Heat Transfer, and Thermodynamics.
3. Plant Systems Design, Control and Instrumentation.
4. Plant Protection Systems.
5. Engineered Safeguards Systems.
6. Fuel Handling and Core Parameters.
7. Mitigating Core Damage.
8. Radiation Protection and Control.
9. General and Specific Plant Operating Characteristics.

Technical Specifications and Title 10, Code of Federal Regulations shall be included in lectures as applicable.

4.2.1.2 Lecture Series Topic Selection

The topics presented in the Fundamental Review series should reflect the general results of the annual examinations and performance of the licensed personnel as evaluated by their supervisors.

The scope of the lecture series should be determined by the Director, Nuclear Training. The depth of coverage in each topic should reflect typical deficiencies identified by the annual examinations.

4.2.2 Operational Proficiency Lecture Series

The Operational Proficiency training sessions cover areas which involve essential plant operational guidelines.

4.2.2.1 Lecture Series Topics

The Operational Proficiency lecture topics are selected to ensure coverage of essential plant operational guidelines and to ensure operational changes and experiences are integrated into licensed individual's training. The lecture topics include the following:

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1. Administrative Procedures, Conditions and Limitations
2. Major Operational Evolutions
3. Facility Design and License Changes
4. Procedures - Normal, Abnormal, Emergency Operating, and Radiological Control
5. Operating History and Problems
6. Related Nuclear Industry Operating Experience
7. Procedure Changes
8. Reportable Occurrences

4.2.2.2 Lecture Series Topic Selection

The topics presented in the Operational Proficiency series will include all the topics listed in Section 4.2.2.1, unless the applicable information is covered in another appropriate manner, such as on-shift discussion sessions, staff meetings, or written communications to each licensed individual, and documented in accordance with procedures NCT-104, "Training Records" or ODA-106, "Review of Documents".

The scope of the lecture series should be determined by the Director, Nuclear Training taking into account the licensed individual's Supervisor's input, the general results of the annual examinations, and the available operational experience inputs.

4.2.3 Pre-Planned Lecture Series Schedule

The requalification training program will be conducted on a two year cycle.

Lectures should be evenly spaced throughout the year except during heavy vacation and refueling periods. Lectures may be deferred due to unanticipated events, but should be conducted as soon as practicable thereafter and within the two-year cycle.

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4.2.4 Pre-Planned Lecture Series Attendance

Licensed individuals shall attend:

1. All pre-planned lectures relating to any annual examination subject in which they received a grade of less than 80% on the previous examination.

Note: Attendance for all cold licensed personnel shall be based upon their scores on the USNRC exam.

2. All Operational Proficiency Lectures regardless of their scores on the previous annual examination or USNRC examination.
3. Any Fundamental Review Lecture if so directed by their supervisor.

4.2.5 Training Aids

Training aids such as films, video tapes, and slides may be used and some self-study may be required in conjunction with the lectures. An instructor will participate in a least fifty percent of the lecture series.

4.3 On-The-Job Training

4.3.1 Control Manipulations

Reactor operators shall manipulate and senior reactor operators shall manipulate or direct the activities of those manipulating the station controls during the term of their licenses for the following activities. Items 1 through 6 shall be performed annually, all other items shall be performed on a two-year basis. The control manipulations which are not performed at the plant shall be performed on a suitable simulator.

1. Manual control of steam generators and/or feedwater during startup and shutdown.
2. Any significant power changes in manual rod control (>10%).
3. Loss of coolant including:
 - a. Significant steam generator leaks.
 - b. Inside and outside primary containment.

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- c. Large and small, including leak-rate determination.
- d. Saturated reactor coolant response.
- 4. Loss of core coolant flow/natural circulation.
- 5. Loss of all feedwater (normal and emergency).
- 6. Plant and reactor startup to the point of adding heat and heatup rate established.
- 7. Plant shutdown.
- 8. Boration and/or dilution during power operation.
- 9. Loss of electrical power (and/or degraded power sources).
- 10. Loss of condenser vacuum.
- 11. Loss of station service water.
- 12. Loss of shutdown cooling (Residual Heat Removal).
- 13. Loss of component cooling water system or cooling to an individual component.
- 14. Loss of normal feedwater or normal feedwater system failure.
- 15. Loss of protective system channel (inverter).
- 16. Mispositioned control rod(s) (or rod drops).
- 17. Inability to drive control rods.
- 18. Conditions requiring use of emergency boration.
- 19. Fuel cladding failure or high activity in reactor coolant or offgas.
- 20. Turbine or generator trip.
- 21. Malfunction of automatic control system(s) which affect reactivity.
- 22. Malfunction of reactor coolant pressure/volume control system.
- 23. Reactor trip.

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24. Main steam line break (inside or outside containment).
25. Nuclear instrumentation failure(s).
26. Loss of instrument air (when simulated on a plant specific simulator).

In the event that an actual abnormal/emergency condition occurs at the plant and performance of the licensed personnel controlling the condition is satisfactory (as evaluated by their Supervisor), credit for completion of a training exercise may be taken.

A record of control manipulations shall be maintained on Form TRA-204-1, "Requalification Control Manipulations" (Attachment 1). A notebook in the Shift Supervisor's office shall be used to contain all forms in use for the current requalification period (as defined by the Operations Supervisor). All forms shall be reviewed by the Operations Supervisor and then transmitted to the Director, Nuclear Training upon expiration of the scheduled completion date.

If personnel do not complete the required control manipulations within 3 months prior to the scheduled completion date, they may be scheduled at a simulator to obtain the required control manipulations. Observation and evaluation of performance during simulated abnormal and emergency conditions will be conducted by simulator training personnel.

4.3.1.1 Simulator Selection

The plant specific CPSES control room simulator will be used when available. When the CPSES simulator is not available, the simulator selected for training use will reproduce the general operating characteristics of CPSES, and the arrangement of the instrumentation and controls of the simulator will be similar to that of CPSES.

4.3.2 Knowledge of Plant Systems

Reactor operators and senior reactor operators shall have an understanding of the operation of controls and equipment and shall be familiar with the operating procedures in each area for which they are licensed. Frequent manipulation of the controls and use of plant operating procedures is expected to maintain the operator proficient during the normal conduct of his duties.

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4.3.3 Review of Abnormal and Emergency Procedures

Reactor operators and senior reactor operators shall review the contents of Abnormal Conditions Procedures and Emergency Response Guidelines annually, plus or minus three months. Verification of review will be provided by sign-off sheets. (Attachment 2). All completed forms shall be reviewed by the Operations Supervisor and then transmitted to the Director, Nuclear Training upon expiration of the scheduled completion date. A review of Abnormal Conditions Procedures and Emergency Response Guidelines shall also be conducted as part of the regular scheduled lecture series (Section 4.2.2.1) and the annual examination (Section 4.4.1).

In addition, a review of the Abnormal Conditions Procedures and Emergency Response Guidelines shall be performed by performance evaluations (Section 4.4.3), on-shift walkthroughs of the procedures conducted by the Shift Supervisor or Assistant Shift Supervisor (and documented per NOT-104, "Training Records"), and actual use on a simulator (Section 4.3.1) conducted by training personnel.

4.4 Evaluation

4.4.1 Annual Examination

In order to determine each licensed individual's knowledge of topics covered in the requalification program and provide a basis for determining areas in which retraining is needed, an annual written requalification examination shall be administered to each licensed reactor operator and senior reactor operator during each calendar year. Separate reactor operator and senior reactor operator examinations shall be administered. This annual examination shall cover the following subjects with depth appropriate to the license:

1. Principles and Theory of Nuclear Power Plant Operation.
2. Fundamentals and Theory of Fluid Flow, Heat Transfer, and Thermodynamics.
3. Plant Systems Design, Control and Instrumentation.
4. Engineered Safeguards Systems.
5. Procedures - Normal, Abnormal, Emergency Operating, and Radiological Control.

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6. Radiation Protection and Control.
7. Tech. Specs. and Applicable 10CFR Sections.
8. Mitigating Core Damage
9. Administrative Procedures, Conditions, and Limitations.
10. Fuel Handling and Core Parameters.
11. Plant Protection Systems.
12. General and Specific Plant Operating Characteristics.

Licensed personnel scoring less than 80% on any category of the most recent written examination will be required to attend the pre-planned lectures applicable to that section. Licensed personnel scoring less than 70% in any category or less than 80% overall on the most recent written examination will be placed in an accelerated requalification training program (Section 4.5.1).

A maximum of three individuals responsible for preparing and administering the examination will be credited with successfully completing the examination.

4.4.2 Periodic Examination

After each lecture or group of lectures, trainees may take a written quiz covering the lecture topic(s). These tests are to evaluate training effectiveness.

A performance standard of 80% will be established for a written quiz. Individuals who receive a grade of less than 80% in a subject may be assigned additional retraining in that subject. The extent of this retraining shall be determined by the Director, Nuclear Training.

4.4.3 Performance Evaluations

Annual performance evaluations of reactor operators and senior reactor operators shall be conducted by the immediate supervisors. These evaluations should include the following areas:

- (a) Shift Relief.
- (b) Log and Record Keeping.
- (c) Plant Awareness and Control.

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- (d) Administrative Procedures.
- (e) Technical Specifications.
- (f) Normal Procedures.
- (g) Emergency Procedures.
- (h) Attitude

Unsatisfactory performance evaluations may result in the licensed individual being placed in the accelerated requalification program (Section 4.5.1).

The Operations Supervisor is responsible for conducting the evaluations of the Shift Supervisors. The Shift Supervisors are responsible for evaluating their subordinate personnel. The appropriate Supervisor shall complete the "Watchstanding Evaluation" form (Attachment 3) and then route it to the Operations Supervisor. The Operations Supervisor and the Operations Superintendent shall then review and sign the form and submit it to the Director, Nuclear Training in accordance with Section 4.7. Non-shift personnel shall be evaluated during simulator retraining. Documentation shall be obtained that includes evaluated areas described in this section and shall be reviewed by the Operations Superintendent and Director, Nuclear Training as a minimum.

4.5 Special Retraining Programs

4.5.1 Accelerated Requalification Program

The accelerated requalification program is for licensed individuals having identified deficiencies requiring assignment to a special retraining effort. Accelerated Requalification Programs shall be assigned based on:

- (a) Unsatisfactory performance (>80% overall) on the annual examination.
- (b) Unsatisfactory performance (>70% on any category) on the annual examination.
- (c) Review of on-the-job proficiency and performance.
- (d) Review of needs for inactive status retraining described in Section 4.5.2.

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If an individual is enrolled in the accelerated requalification training program, he shall not perform licensed duties involving the area in which he is deficient until he has successfully completed the accelerated program.

These programs should be tailored to the needs of the individual. The Director, Nuclear Training will tailor the scope and duration of the accelerated program to the individual's demonstrated deficiencies. The individual's supervisor should provide any appropriate recommendations to the Director, Nuclear Training regarding program content.

The program content should be specifically structured to upgrade knowledge and skills identified as deficient. Examination categories and areas in which performance standards were not met shall be covered in the program.

Successful completion of the accelerated requalification program shall be determined by administering an examination. The examination shall cover all categories of the requalification written examination in which the individual was originally deficient. Performance standards for the accelerated requalification program shall be: a score of at least 80% on each written examination category or satisfactory completion of an oral examination.

In the event that these standards are not met, the individual's suitability for resuming licensed duties will be reviewed by the Director, Nuclear Training. The Director, Nuclear Training shall provide a recommendation to the Operations Superintendent regarding the individual's status in relation to licensed duties or additional upgrading efforts to be considered. If appropriate, another accelerated requalification program shall be structured to correct deficiencies.

4.5.2 Inactive Status Retraining

If a licensed individual has not actively carried out licensed duties for a period in excess of four months, a special retraining program or evaluation is required prior to resuming licensed duties.

Active status can be maintained by performance of licensed duties and participation in this Licensed Operator Requalification Training Program.

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In the event that a licensed individual does not maintain an active status, he may be required to attend specific pre-planned lecture series or to take a written examination similar in scope and format to the annual written examination (Section 4.4.1) prior to resuming licensed duties. Consideration may be given to assigning the licensed individual to a training status on an operating shift prior to resuming licensed duties.

The performance standards applied to the annual requalification examination shall be used in evaluating the results of this written examination. If the performance standards are not met, the licensed individual shall complete an accelerated requalification program prior to resuming licensed duties.

4.5.3 Newly Licensed Individuals

Newly licensed individuals shall enter the requalification program and participate in the annual program cycle upon receipt of their license. Newly licensed individuals successfully completing their NRC licensing examination less than six (6) months prior to an annual requalification examination may be excused from taking the current annual written examination.

4.6 Staff Members

Individuals who maintain reactor operator or senior reactor operator licenses for the purpose of providing backup capability to the operating staff shall participate in the requalification program except to the extent that their normal duties preclude the need for specific retraining in particular areas.

As a minimum these individuals shall:

- (a) Take the annual written examination and participate in the lecture series based on the results thereof. (Sections 4.2.1 and 4.4.1)
- (b) Manipulate the controls or supervise the manipulation of the controls as specified in Section 4.3.1.
- (c) Systematically review design changes, procedure changes and facility license changes. (Section 4.2.2)
- (d) Systematically review the content of Abnormal Conditions Procedures and Emergency Response Guidelines on a regularly scheduled basis. (Section 4.3.3)
- (e) Receive performance evaluations described in Section 4.4.3.

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4.7 Records

Records of the Licensed Operator Requalification Training Program shall be maintained in an auditable manner. The Director, Nuclear Training is responsible for maintaining the following records:

- (a) Copies of the written examinations administered, and the answers given, results of evaluations, and documentation of any training administered pursuant to Section 4.5.1 and 4.4.2.
- (b) Annual performance evaluations
- (c) Additional training administered in areas in which an operator or senior operator has exhibited deficiencies.
- (d) Annual review of abnormal and emergency procedures.
- (e) Control manipulation and plant evolution participation records.

All training records specified by this procedure shall be maintained in accordance with procedure NOT-104, "Training Records".

4.8 References

- 4.8.1 FSAR, Section 13.2.2.1
- 4.8.2 INPO Guideline GPG 02-10-80, "Nuclear Power Plant Requalification Program For Licensed Personnel".
- 4.8.3 10 CFR Part 55, Appendix A - Requalification Programs for Licensed Operators of Production and Utilization Facilities.
- 4.8.4 ANSI 18.1 - 1971, "Selection and Training of Nuclear Power Plant Personnel".
- 4.8.5 ANS 3.1 - 1980, "Standard for Selection, Qualification, and Training of Personnel for Nuclear Power Plants".
- 4.8.6 CPSES Procedure NOT-104, "Training Records".
- 4.8.7 USNRC Letter Dated March 28, 1980, "Qualifications of Reactor Operators" (Harold Denton letter).
- 4.8.8 USNRC Letter dated December 16, 1981, "Qualification of Reactor Operators - License Examinations" (Darrell G. Eisenhut letter).

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5.0 Attachments

- 5.1 Requalification Control Manipulation, Attachment 1
- 5.2 Requalification Program Emergency/Abnormal Procedures Checklist, Attachment 2
- 5.3 Watchstanding Evaluation, Attachment 3

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ATTACHMENT 1
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REQUALIFICATION CONTROL MANIPULATIONS

NAME: _____ LICENSE: RO SRO
(Please Print) (Circle One)

SCHEDULED COMPLETION DATE: _____

Type of Control Manipulation	Shift: Night/Day/Evening	Unit: 1/2	Date	S.S. Initial
A. Manual control of steam generators and/or feed-water during startup and shutdown	_____	_____	_____	_____
B. Any significant power changes in manual rod control (> 10%)	_____	_____	_____	_____
C. Loss of coolant:				
1. Significant steam generator leak	_____	_____	_____	_____
2. Inside/outside primary containment	_____	_____	_____	_____
3. Large and small, including leak-rate determination	_____	_____	_____	_____
4. Saturated reactor coolant response	_____	_____	_____	_____
D. Loss of core coolant flow/natural circulation	_____	_____	_____	_____
E. Loss of all feedwater (normal and emergency)	_____	_____	_____	_____
F. Plant and Reactor Startup to the point of adding heat	_____	_____	_____	_____
G. Plant shutdown	_____	_____	_____	_____

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REQUALIFICATION CONTROL MANIPULATIONS

NAME: _____ LICENSE: RO SRO
(Please Print) (Circle One)

SCHEDULED COMPLETION DATE: _____

Type of Control Manipulation	Shift: Night/Day/Evening	Unit: 1/2	Date	S.S. Initial
H. Boration and/or dilution during power operation	_____	_____	_____	_____
I. Loss of electrical power (and/or degraded power sources)	_____	_____	_____	_____
J. Loss of condenser vacuum	_____	_____	_____	_____
K. Loss of station service water	_____	_____	_____	_____
L. Loss of shut down cooling	_____	_____	_____	_____
M. Loss of component cooling water system or cooling to an individual component	_____	_____	_____	_____
N. Loss of normal feedwater or normal feedwater system failure	_____	_____	_____	_____
O. Loss of protective system channel (inverter)	_____	_____	_____	_____
P. Mispositioned control rod(s) (or rod drops)	_____	_____	_____	_____
Q. Inability to drive control rods	_____	_____	_____	_____

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REQUALIFICATION CONTROL MANIPULATIONS

NAME: _____ LICENSE: _____ RO _____ SRO _____
(Please Print) (Circle One)

SCHEDULED COMPLETION DATE: _____

	Type of Control Manipulation	Shift: Night/Day/Evening	Unit: 1/2	Date	S.S. Initial
R.	Conditions requiring use of emergency boration	_____	_____	_____	_____
S.	Fuel cladding failure or high activity in reactor coolant or offgas	_____	_____	_____	_____
T.	Turbine or generator trip	_____	_____	_____	_____
U.	Malfunction of automatic control system(s) which affect reactivity	_____	_____	_____	_____
V.	Malfunction of reactor coolant pressure/volume control system	_____	_____	_____	_____
W.	Reactor trip	_____	_____	_____	_____
X.	Main steam line break (inside or outside containment)	_____	_____	_____	_____
Y.	Nuclear instrumentation failure(s)	_____	_____	_____	_____
Z.	Loss of instrument air	_____	_____	_____	_____

Operations Supervisor Review: /s/ _____ Date: _____

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REQUALIFICATION PROGRAM
EMERGENCY/ABNORMAL PROCEDURES CHECKLIST

NAME: _____ SCHEDULED COMPLETION DATE: _____
(Please Print)

<u>Procedure No.</u>	<u>Title</u>	<u>Signature/Date</u>
EOP-0.0	Reactor Trip or Safety Injection	_____
EOS-0.0	Reactor Trip Recovery	_____
EOS-0.2	Natural Circulation Cooldown	_____
EOS-0.3	SI Termination Following Spurious SI	_____
EOS-0.4	Natural Circulation Cooldown with Steam Void in Vessel Upper Head	_____
EOP-1.0	Loss of Reactor Coolant (LOCA)	_____
EOS-1.1	SI Termination Following LOCA	_____
EOS-1.2	Post-LOCA Cooldown and Depressurization	_____
EOS-1.3	Transfer to Cold Leg Recirculation Following LOCA	_____
EOS-1.4	Transfer to Hot Leg Recirculation	_____
EOP-2.0	Loss of Secondary Coolant (LOSC)	_____
EOS-2.1	SI Termination Following LOSC	_____
EOS-2.2	Transfer to Cold Leg Recirculation Following LOSC	_____
EOP-3.0	Steam Generator Tube Rupture (SGTR)	_____
EOS-3.1	SI Termination Following SGTR	_____

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REQUALIFICATION PROGRAM
EMERGENCY/ABNORMAL PROCEDURES CHECKLIST

NAME: _____
(Please Print)

<u>Procedure No.</u>	<u>Title</u>	<u>Signature/Date</u>
EOS-3.2	SGTR Alternate Cooldown By Back-filling RCS	_____
EOS-3.3	SGTR with Secondary Depressurization	_____
ECA-1.0	Anticipated Transient Without TRIP (ATWT)	_____
ECA-2.0	Loss of All AC Power	_____
ECA-2.1	Loss of All AC Power Recovery Without SI Required	_____
ECA-2.2	Loss of All AC Power Recovery With SI Required	_____
ECA-3.0	SGTR Contingencies	_____
ECA-4.0	Response To Multiple Steam Generator Depressurization	_____
ECA-5.0	Loss of Emergency Coolant Recirculation	_____
ECA-6.0	Secondary High Energy Line Rupture with Loss of SI Function	_____
ECA-7.0	Combined SGTR and LOCA	_____
ECA-8.0	Unisolable SGTR	_____

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REQUALIFICATION PROGRAM
EMERGENCY/ABNORMAL PROCEDURES CHECKLIST

NAME: _____
(Please Print)

<u>Procedure No.</u>	<u>Title</u>	<u>Signature/Date</u>
ECA-9.0	SGTR Without Pressurizer Pressure Control	_____
FRS-0.1	Response to Nuclear Power Generation	_____
FRS-0.2	Response to Loss of Core Shutdown	_____
FRC-0.1	Response to Inadequate Core Cooling	_____
FRC-0.2	Response to Degraded Core Cooling	_____
FRC-0.3	Response to Potential Loss of Core Cooling	_____
FRC-0.4	Response to Saturated Core Cooling Conditions	_____
FRP-0.1	Response to Imminent Pressurized Thermal Shock Conditions	_____
FRP-0.2	Response to Anticipated Pressurized Thermal Shock Conditions	_____
FRH-0.1	Response to Loss of Secondary Heat Sink	_____
FRH-0.2	Response to Steam Generator Overpressure	_____
FRH-0.3	Response to Steam Generator High Level	_____
FRH-0.4	Response to Steam Generator Low Level	_____

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REQUALIFICATION PROGRAM
EMERGENCY/ABNORMAL PROCEDURES CHECKLIST

NAME: _____
(Please Print)

<u>Procedure No.</u>	<u>Title</u>	<u>Signature/Date</u>
FRH-0.5	Response to Loss of Steam Generator PORVs and Condenser Dump Valves	_____
FRZ-0.1	Response to Containment High Pressure	_____
FRZ-0.2	Response to High Containment Sump Level	_____
FRZ-0.3	Response to High Containment Radiation Level	_____
FRI-0.1	Response to Pressurizer Flooding	_____
FRI-0.2	Response to Low System Inventory	_____
FRI-0.3	Response to Voids in Reactor Vessel	_____
ABN-101	Reactor Coolant Pump Malfunctions	_____
ABN-102	Excessive Reactor Coolant Leakage	_____
ABN-103	High Reactor Coolant Leakage	_____
ABN-104	Residual Heat Removal System Malfunction	_____
ABN-105	Chemical and Volume Control System Malfunction	_____
ABN-106	Abnormal Containment Conditions	_____
ABN-108	Steam Generator Tube Leakage	_____

TRA-204-2c

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REQUALIFICATION PROGRAM
EMERGENCY/ABNORMAL PROCEDURES CHECKLIST

NAME: _____
(Please Print)

<u>Procedure No.</u>	<u>Title</u>	<u>Signature/Date</u>
ABN-109	Pressurizer Pressure/Level Control Malfunction	_____
ABN-201	Recovery From a Spurious Phase B Actuation	_____
ABN-202	Recovery From a Spurious Main Steam Isolation	_____
ABN-301	Steam Dump System Malfunction	_____
ABN-302	Feedwater/Condensate System Malfunction	_____
ABN-303	Auxiliary Feedwater System Malfunction	_____
ABN-304	Condenser Tube Leakage	_____
ABN-305	Circulating Water System Malfunction	_____
ABN-401	Turbine Malfunction	_____
ABN-402	Main Generator Malfunction	_____
ABN-501	Station Service Water System Malfunction	_____
ABN-502	Component Cooling Water System Malfunction	_____
ABN-503	Spent Fuel Pool Cooling and Cleaning System Malfunction	_____

TRA-204-2d

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REQUALIFICATION PROGRAM
EMERGENCY/ABNORMAL PROCEDURES CHECKLIST

NAME: _____
(Please Print)

<u>Procedure No.</u>	<u>Title</u>	<u>Signature/Date</u>
ABN-601	Operations During Abnormal System Transmission Conditions	_____
ABN-701A	Source Range Instrumentation Malfunction	_____
ABN-702A	Intermediate Range Instrumentation Malfunction	_____
ABN-703	Power Range Instrumentation Malfunction	_____
ABN-70	T _c /N-16 Instrumentation Malfunction	_____
ABN-705	Pressurizer Pressure Instrumentation Malfunction	_____
ABN-706	Pressurizer Level Instrumentation Malfunction	_____
ABN-707	Steam Flow Instrumentation Malfunction	_____
ABN-708	Feedwater Flow Instrumentation Malfunction	_____
ABN-709	S/G Pressure, Steam Line Pressure, P _{imp} Instrumentation Malfunction	_____
ABN-710	Steam Generator Level Instrumentation Malfunction	_____
ABN-711	Inverter Malfunction	_____
ABN-901	Fire Protection System Malfunctions	_____
ABN-902	Accidental Release of Radioactive Gas	_____

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REQUALIFICATION PROGRAM
EMERGENCY/ABNORMAL PROCEDURES CHECKLIST

NAME: _____
(Please Print)

<u>Procedure No.</u>	<u>Title</u>	<u>Signature/Date</u>
ABN-903	Accidental Release of Radioactive Liquid	_____
ABN-904	Accidental Release of Chlorine Gas	_____
ABN-905	Loss of Control Room Habitability	_____
ABN-906	Loss of P2500 Computer	_____
ABN-907	Acts of Nature	_____
ABN-908	Fuel Handling Accident	_____

Operations Supervisor Review: /s/ _____ Date: _____

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WATCHSTANDING EVALUATION

I. SUPERVISOR REVIEW

1. Has this individual caused or been responsible for a significant operating event or reportable occurrence?
2. Has this individual compromised the safety of the plant or the public during this review period?

If answer to 1 or 2 is yes, explain. (attach addition sheets if necessary)

3. Is this individual a safe operator?

YES	NO
X	X

J. ADDITIONAL COMMENTS

Provide additional information for area above indicated as unsat.

Supervisor(s) _____ Date _____

Operations Supervisor Review _____

Operations Superintendent Review _____

TNRC-83011

TEXAS UTILITIES GENERATING COMPANY

2001 BRYAN TOWER - DALLAS, TEXAS 75201

August 23, 1983

BILLY R. CLEMENTS
VICE PRESIDENT

50-445

Mr. Harold R. Denton
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Denton:

The Comanche Peak Steam Electric Station Licensed Operator
Requalification Training Program Description (CPSES Procedure
TRA-204, Revision 3) is attached for NRC approval.

The program has been revised to incorporate the changes
detailed in your review dated July 12, 1983. We would appreciate
approval of this training program by September 19, 1983, so that
it can be implemented following receipt of operator licensing
examination results.

Sincerely,

Billy R. Clements

BRC:jac

Attachment

cc: J. C. Kuykendall
R. A. Jones
R. B. Seidel
L. G. Barnes
C L Turner
R. W. Haskovec

ARMS

M0003
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