

U.S. NUCLEAR REGULATORY COMMISSION  
LICENSEE EVENT REPORTAPPROVED BY OMR  
3130-0011CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)01 ALBRF 2200-000000-000341111145  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100CONT  
01 REPORT SOURCE L605000026070725830823830  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 During normal operation, while performing Surveillance Instruction 4.2.B-60  
03 it was found that the Core Spray NE Room Cooler fan would not operate  
04 (Technical Specification Table 3.5.D-2). There was no effect on public  
05 health and safety. Technical Specification 3.5.D.2 requires the CS pumps  
06 be declared inoperable when the area coolers are inoperable. This placed  
07 the unit in a 7 day LCO. The redundant core spray system loop and room  
08 cooler were operable.

09 SYSTEM CODE S F 11 CAUSE CODE E 12 CAUSE SUBCODE A 13 COMPONENT CODE RELAYX 14 COMP SUBCODE D 15 VALVE SUBCODE Z 16  
17 LEAD REPORT NUMBER 83 18 SEQUENTIAL REPORT NO 049 19 OCCURRENCE CODE 03 20 REPORT TYPE L 21 REVISION NO 0  
ACTION TAKEN C 19 FUTURE ACTION Z 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 00000 22 ATTACHMENT SUBMITTED Y 23 NPD-1 FORM SUM Y 24 PRIME COMP. SUPPLIER L 25 COMPONENT MANUFACTURER G080

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The overload relays in the GE type CR106CO full-voltage non-reversing  
11 magnetic starter tripped. The overload relay, GE model number CR124C028,  
12 was replaced and motor current verified to be acceptable. This is  
13 considered a random failure and no further recurrence control is required.

14  
15 FACILITY STATUS E 28 % POWER 095 29 OTHER STATUS NA 30 METHOD OF DISCOVERY B 31 DISCOVERY DESCRIPTION Surveillance tests 32

16 ACTIVITY CONTENT RELEASED OF RELEASE Z 33 AMOUNT OF ACTIVITY NA 34 LOCATION OF RELEASE NA 35

17 PERSONNEL EXPOSURES NUMBER 000 37 TYPE Z 38 DESCRIPTION NA 39

18 PERSONNEL INJURIES NUMBER 000 40 DESCRIPTION NA 41

19 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43

20 PUBLICITY ISSUED N 44 DESCRIPTION 45

8308300523 830823  
PDR ADOCK 05000260  
S PDR

NRC USE ONLY

NAME OF PREPARER W. T. Christopher

PHONE (205) 729-0797

LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 83049 Technical Specification Involved Table 3.5.D-2

Reported Under Technical Specification 6.7.2.b(2) \* Date Due NRC 8/24/83

Event Narrative:

Unit 1 was in a refueling outage, Unit 2 was operating normally at 95 percent power, Unit 3 was operating normally at 87 percent power. Unit 2 was the only unit affected by this event. On July 25, 1983, while performing Surveillance Instruction 4.2.B-60 (Instrumentation That Initiate or Control the CSCS Thermostat (Core Spray Area Cooler Fan)) it was discovered that the Core Spray NE Room Cooler Fan would not operate. Investigation revealed that the overload relay, GE model number CR124C028, on the GE type CR106C0 full voltage non-reversing motor starter had tripped. The overload relay was replaced and the motor current was verified to be acceptable. The core spray NE room cooler fan was out of service for 8 hours. Technical Specification Table 3.2.B allows 24 hours to repair before declaring the system inoperable. However T.S 3.5.D-2 requires the system be declared inoperable if area cooler is not operable. This places the unit in a 7 day LCO. There was no effect on public health and safety. The redundant core spray system loop and room cooler were operable. This is considered a random failure and no further recurrence control is required.

\* Previous Similar Events:

BFRO-50-260/8147, 8163

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

\*Revision: JRP

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

USNRC REGION II  
ATLANTA, GEORGIA

83 AUG 25 49:08

August 23, 1983

Mr. James P. O'Reilly, Director  
U.S. Nuclear Regulatory Commission  
Suite 2900  
101 Marietta Street, NW  
Atlanta, Georgia 30303

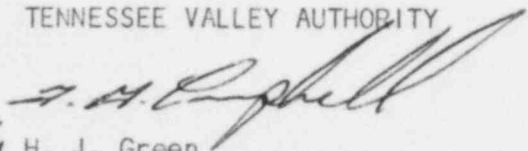
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET  
NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE  
REPORT BFR0-50-260/83049

The enclosed report provides details concerning overload relays found  
tripped. This report is submitted in accordance with Browns Ferry  
unit 2 Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

  
H. J. Green  
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

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Institute of Nuclear Power Operations  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

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