MEMORANDUM FOR: D. G. Eisenhut, Director, Division of Licensing, NRR

FROM: C. E. Norelius, Director, Division of Project and Resident Programs, Region III

SUBJECT: COMMONWEALTH EDISON COMPANY, LASALLE COUNTY STATION UNIT 2, STATUS OF PREPAREDNESS FOR LICENSING (UPDATE)

Reference: C. E. Norelius memorandum to D. G. Eisenhut dated July 25, 1983.

This memorandum is forwarded in accordance with IE MC94300 to update your information on the preparedness for licensing of LaSalle County Station, Unit 2. Enclosures 1 through 4 include items requiring inspection and resolution before a finding of readiness for operation can be endorsed by Region III. The categories used are consistent with those used by the applicant.

A meeting between the applicant, NRR and Region III was held on August 3, 1983, in Bethesda, Maryland. The purpose of the meeting was to discuss the applicant's proposed preoperational test/system demonstration deferrals and other issues pertinent to operational preparedness. Enclosure 5 includes a memorandum from the Senior Resident Inspector (SRI) at LaSalle documenting the information presented and conclusions reached during that meeting. Additionally, the SRI has documented two conflicts between the proposed deferrals and anticipated Unit 2 technical specifications. The status of those preoperational tests/system demonstrations is included in Enclosure 2.

We maintain our belief that the applicant's projected mid-September fuel load date is optimistic considering the preoperational testing remaining, the number of system deficiencies requiring resolution, and the number of NRR outstanding open items. The significance of these and other outstanding items, relative to the schedule for operating license issuance, is addressed in the enclosures.

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If you have any questions concerning this matter, please contact either Mr. R. D. Walker or Mr. W. G. Guldemond of my staff.

"Original Signed by C.E. Norelius"

C. E. Norelius, Director
Divisions of Project and
Resident Programs

Enclosures:
1. Outstanding Items including Enforcement and Unresolved Items
2. Preoperational Testing and Operational Preparedness Status
3. Construction Status
4. Status of Inspections required by MC 2512, MC 2513, and MC 2514 (Summary)
5. W. G. Guldemond Memorandum to R. D. Walker dated August 5, 1983

cc w/encls:
J. Taylor, IE
E. Jordan, IE
A. Bournia, NRR

RIII
Guldemond
8/19

RIII	RIII	RIII	RIII	RIII	RIII	RIII
Grobe	Walker	Shafer	Knop	Hind	Spessard	Norelius
8/18 08/18/83	8/19	8/19	8-1983	8/19/83	8/19	8/19/83

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ENCLOSURE 1

OUTSTANDING ITEMS INCLUDING ENFORCEMENT AND UNRESOLVED ITEMS

I. Enforcement Items

a. Items Requiring Resolution By Fuel Load (Category 1).

#1. Failure to develop adequate procedures for or implement procedures for calibration of measuring and test equipment. (374/82-10-05(DE))

#2. Failure to adequately control nonconforming measuring and test equipment. (374/82-10-06(DE))

#3. Failure to complete nondestructive examinations required by design drawings prior to accepting work. (374/82-10-07(DE))

#4. Failure to implement revisions to procedures controlling work which is quality related. (374/82-10-09(DE))

#5. Failure to adequately implement voltage/amperage welding surveillance procedure. (374/82-10-10(DE)).

#6. Failure to produce and maintain adequate records for measuring and test equipment including receipt inspections. (374/82-10-11(DE))

#7. Failure to include certain rework items in nondestructive examination control procedures. (374/82-10-12(DE))

#8. Failure to conduct adequate audits. (374/82-10-13(DE))

#9. Failure to implement the requirements for Level I, II, and III inspections as delineated in ANSI N45.2.6. (374/82-10-15(DE))

#10. Failure to verify that excess flow check valve testing requirements were properly incorporated into preoperational tests prior to deleting the requirements from another test. (374/83-05-01(DE))

#11. Failure to properly review revisions to a preoperational test which changed the intent of the test. (374/83-05-02(DE))

#12. Failure to follow requirements for disassembly of completed pipe support restraints. (374/82-24-01(DE))

#The licensee has informed Region III that the necessary work to resolve these items is complete and ready for Region III inspection.

b. Items That Do Not Need to be Resolved Prior to Fuel Load, Initial Criticality, or Full Power Operation (Category 2).

1. Failure to establish prerequisites prior to performance of a preoperational test. (374/83-23-01(DE))

2. Failure to follow the containment integrated leak rate test procedure. (374/83-23-03(DE))

3. Failure to tag valves during the containment integrated leak rate test procedure. (374/83-23-04(DE))

c. Items That Must Be Resolved By the First Refueling Outage (Category 3)

#1. Failure to establish a housekeeping policy for fire stops. (374/82-22-03(DE))

#2. Inadequate control of fire stop installations. (374/82-22-01(DE))

d. Items That Must be Resolved Prior to Initial Criticality (Category 4).

#1. Inadequate and technically incorrect surveillance procedures for turbine control valve position reactor protection system functions. (374/83-06-01(DE))

e. Items That Must Be Resolved Prior to Power Operation or Full Power Operation as Applicable (Category 5).

1. None at this time.

II. Unresolved and Open Items

a. Items Required by Fuel Load (Category 1).

1. Conduct tests to confirm the flow rate of the control rod drive system with two pump operation. (374/81-00-07(DPRP))

#2. Perform a containment high pressure bypass leakage test. (374/81-00-08(DPRP))

3. Perform a secondary containment leak test. (374/81-00-10(DPRP))

#4. Perform hydrostatic testing of emergency core cooling system suction valves, relief valve discharge lines, and pump test and minimum flow valves, and reactor head spray valves. (374/81-00-12(DPRP))

#5. Perform tests that demonstrate that overriding or resetting engineered safety features actuation signals does not cause any equipment to change position. (374/81-00-13(DPRP))

6. Change the power supply for the rod position information system to an uninterruptable power supply. (374/81-00-14(DPRP))

#7. Install qualified position indication switches on main steam relief valves. (374/81-00-15(DPRP))

#8. Install dedicated hydrogen penetrations. (374/81-00-16(DPRP))

#9. Ensure all engineered safety feature systems are aligned per approved mechanical and electrical checklists. (374/81-00-17 (DPRP))

#10. Event V check valve resolution. (374/81-00-18(DPRP))

#11. Failure to maintain an up-to-date list of contractor generated documents. (374/82-10-01(DE))

#12. Inadequate traveler package documentation. (374/82-10-02(DE))

#13. Suspect quality assurance documentation. (374/82-10-03(DE))

#14. Photocopied quality control acceptance signatures. (374/82-10-04(DE))

#15. Sufficient number of deviations from code requirements. (374/82-10-08(DE))

#16. Contractors were informed of impending audits. (374/82-10-14(DE))

#17. Lessons learned during Unit 1 preoperational testing were not adequately incorporated into Unit 2 testing. (374/83-05-04(DE))

#18. Inspect ring lug terminations for containment penetrations per IE Bulletin 82-04. (374/83-10-01(DPRP))

19. Complete review and corrective actions of the surveillance matrix prior to fuel load. (374/83-18-01(DPRP))

20. Lack of instruction for providing marked-up drawings to operating staff. (374/83-13-04 (DE))

21. Replace Rosemount Pressure Transmitters per IE Bulletin 80-16. (374/83-08-01(DE))

22. Improper computer constants for thermal limit calculations. (374/83-23-02(DE))

23. Final review of containment integrated leak rate test results. (374/83-23-05(DE))

#24. Resolve cable separation criteria application. (374/83-22-01(DE))

b. Items That Do Not Need to be Resolved Prior to Fuel Load, Initial Criticality, or Full Power Operation (Category 2).

1. Modify the Reactor Core Isolation Cooling System to autostart on low reactor vessel water level after tripping. (374/81-00-04a(DPRP))

2. Install timers on the Reactor Core Isolation Cooling System steam line isolation logic. (374/81-00-04b(DPRP))

3. Environmental qualification of electrical equipment. (374/81-00-19(DPRP))

4. Post fuel load IE Bulletin 80-11 requirements. (374/83-08-02(DPRP))

5. Resolution of organic contamination of the condensate system. (374/83-16-01(DPRP))

6. G.E. telephone relays installed in protective systems. (374/83-16-03(DPRP))

7. Breakthroughs in fire stops were not controlled. (374/82-22-02(DE))

#8. Requirement for valve stem material certifications not clear. (374/83-13-01(DE))

9. Modification packages did not identify drawings changed. (374/83-13-02(DE))

10. Lack of requirement for documents in modification history. (374/83-13-03(DE))

11. Procedure revisions are required to clarify when the time clock starts for emergency notifications. (374/83-21-01(EPS))

12. Inadequate corporate guidance on GSEP record-keeping. (374/83-21-02(EPS))

13. Inadequate station guidance on GSEP record-keeping. (374/83-21-03(EPS))

14. Inadequate GSEP drill records. (374/83-21-04(EPS))

c. Items That Must be Resolved Prior to Startup Following the First Refueling Outage (Category 3).

1. Provide redundant vent and drain valves and diverse scram discharge volume instrumentation. (374/81-00-03(DPRP))

2. Modify Automatic Depressurization System Logic to eliminate the high drywell pressure trip with a runout timer on low pressure emergency core cooling system actuation. (374/81-00-04c(DPRP))

3. Install instrumentation for an automatic reactor scram during startup and refueling modes on low control rod drive pump discharge pressure. (374/81-00-05(DPRP))

#4. Replace jet pump hold down beams or perform visual and ultrasonic inspections of existing beams. (374/81-00-06(DPRP))

#5. Install debris screens in the containment purge system. (374/81-00-11(DPRP))

6. Install Class IE circuitry that will realign the Reactor Core Isolation Cooling System suction from the condensate storage tank to the suppression pool on low level in the condensate storage tank. (374/81-00-20(DPRP))

7. Install a second level of undervoltage protection designed to meet Class IE requirements. (374/81-00-21(DPRP))

8. Install long term reliability modifications on diesel generators. (374/81-00-22(DPRP))

9. Install indications and alarms for Class IE D.C. power in the control room. (374/81-00-23(DPRP))

#10. Install redundant fault current protection for penetrations. (374/81-00-24(DPRP))

#11. Install diesel generator lube oil system per NUREG/CR-0660. (374/81-00-25(DPRP))

12. Modify High Pressure Core Spray logic to autostart on low water level even if the operator stops the system. (374/81-00-04d(DPRP))

13. Perform destructive testing of a control rod per IE Bulletin 79-26. (374/81-00-26(DPRP))

14. Complete modifications and/or replacement of equipment as a result of fatigue evaluation. (374/81-00-27(DPRP))

15. Provide additional fire detectors. (374/81-00-28(DPRP))

16. Finalize procedures for handling irradiated fuel. (374/83-01-01(DPRP))

#17. Cable density in fire penetration stops were not evaluated against fire test results. (374/82-22-04(DE))

#18. Fire stop material mix proportions were not controlled. (374/82-22-05(DE))

#19. Develop acceptance criteria for fire penetration stops. (374/82-22-06(DE))

d. Items Required to be Resolved Prior to Initial Criticality (Category 4).

#1. Verify that safety related and reactor protection system instrument lines are correctly plumbed and not plugged. (374/83-05-03(DE))

2. Replace main steam line drain socket welds. (374/83-03-01(DE))

#3. Perform a periodic leak test of the main steam isolation valves conducted such that the radiological analysis, which was the basis for the exemption to Appendix J to 10 CFR Part 50, is proven valid. This test must substantiate that the outboard MSIV leak rate is less than 11.5 cubic feet per hour with the inboard MSIV open. (374/81-00-29(DPRP))

4. Control room design reviews of Appendix C of the Safety Evaluation Report. (374/81-00-30(DPRP))

5. Install personnel emergency equipment in the reactor building. (374/83-21-05(EPS))

e. Items That Must be Resolved Prior to Power Operation or Full Power Operation as Applicable.

None.

III. Open Construction Deficiency Reports (10 CFR 50.55(e)) That Must Be Completed by Fuel Load

#1. Static O-Ring switches leak. (374/83-02)

2. Brown-Boveri 4-6 KV switchgear close when charged. (374/83-03)

#3. Deficient Zack weld records. (374/82-01)

#4. Deficient Zack weld records. (374/82-07)

#5. Elcen spring hangers have high carbon nuts welded to low carbon turnbuckles. (374/83-04)

IV. IE Bulletins and Circulars

a. Items Required Prior to Fuel Load.

#1. IE Bulletin 79-28, "Possible Malfunction of Namco Model EA 180 Limit Switches"

2. IE Bulletin 83-03, "Check Valve Failures in Raw Water Cooling Systems of Diesel Generators"

3. IE Bulletin 80-11, "Masonry Walls"- A licensee submittal is required addressing the effects of voids in walls.

b. Items That Do Not Need to be Resolved Prior to Fuel Load, Initial Criticality, or Full Power Operation

1. IE Bulletin 79-11, "Overcurrent Trip Devices."

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ENCLOSURE 2

PREOPERATIONAL TESTING STATUS AND OPERATIONAL PREPAREDNESS

Preoperational Testing Status

Preoperational Testing is approximately 78% complete. The overall status of the test program is summarized below.

Overall Program Status:

	# Tests	# Started	# Completed	# Given Final Approval By CECO	# Reviewed By NRC
Preoperational Tests/ System Demonstrations required for fuel load	72	69	34	26	3
System Demonstrations for low power operation	14	12	8	8	3

Status of Completed Tests Which Have Not Been Reviewed:

	# Tests	Onsite	PED
Preoperational Tests/ System Demonstrations required for fuel load	8	3	5
Systems Demonstrations for low power operation	0	0	0

Preoperational Test/System Demonstrations Required for Fuel Load But Not Started:

Test		Shifts of Testing*
PT-NR-202	Traversing Incore Probe System	14
PT-NR-201B	Intermediate Range Monitors	10
PT-VP-202	Post LOCA Hydrogen Control	1

*This is licensee's estimate of total shifts required to complete list.

System Demonstrations for Low Power Operation Not Started:

Test		Shifts of Testing*
SD-EH-201A	Turbine EHC	20
SD-EH-201B	Turbine Supervisory Panel	18

Preoperational Tests/System Demonstrations Required for Fuel Load But Not Complete:

Test		% Complete	Shifts of Testing*
PT-AP-203	Emergency Power Redundancy	12	15
PT-CM-201	Containment Monitoring	46	7
PT-CM-202	Post Accident Containment Monitoring	23	10
PT-DG-201B	Diesel Generators	92	45
PT-FR-201	Fuel Handling and Vessel Servicing	89	10
PT-IN-201	Drywell Pneumatic	73	14
PT-MS-201A	Main Steam Leakage Control	79	9
PT-MS-201B	MSIV and Instrumentation	82	9
PT-MS-201C	SRV and ADS	4	12
PT-NB-201	Nuclear Boiler	90	10
PT-NR-201A	Source Range Monitor	73	10
PT-NR-201C	LPRM, APRM, RBM	20	16
PT-OG-201	Off Gas System	47	28
PT-PC-201	Primary Containment ILRT	86	50
PT-PC-203	Primary Containment Isolation	58	21
PT-PR-201B	Process Radiation Monitoring	50	13
PT-RD-202	CRD Hydraulic	43	60
PT-RH-201	Residual Heat Removal System	50	50
PT-RI-201	Reactor Core Isolation Cooling System	80	14
PT-RP-202	Remote Shutdown Panel	50	16
PT-RR-201	Reactor Recirculation	74	50
PT-SI-202	Pipe Vibration	91	20
PT-VP-203	Primary Containment HVAC	36	14
PT-VR-201	Reactor Building HVAC	58	28
SD-CX-202	Rod Worth Minimizer	24	5
SD-FC-201	Fuel Pool Cooling	75	10
SD-HY-201	Hydrogen Cooling	84	4
SD-PS-201	Process Sampling	37	28
SD-RT-201	Reactor Water Cleanup	64	34
SD-SA-201	Service and Instrument Air	92	30
SD-SI-201	Dynamic Effects	46	10
SD-TG-201	Turbine Lift Pump	83	7
SD-TO-201	Turbine Lube Oil	27	20
SD-TO-202	Hydrogen Seal Oil	23	10
PT-LD-201	Leak Detection	80	28

System Demonstrations for Low Power Operation Started But Not Completed:

Test		% Complete	Shifts of Testing*
SD-FW-201	Reactor Feedwater	67	14
SD-FW-202	Feedwater Control	37	14
SD-WE-201	Rod Waste Reprocessing	55	15
SD-WX-201	Solid Radwaste	90	10

Systems Released for Operations:

PT-DO-201	Diesel Oil
PT-PR-201A	Process Radiation Monitoring
PT-SC-201	Standby Liquid Control
PT-VD-201	Diesel Generator Ventilation
PT-VP-204	Primary Containment Chilled Water
PT-VX-201	Switchgear Heat Removal
PT-VY-202	CSCS Equipment Ventilation
SD-AS-201	Auxiliary Steam
SD-CD-202	Condenser and Auxiliary Equipment
SD-CY-201	Cycled Condensate
SD-ES-201	Extraction Steam
SD-GC-201	Stator Cooling Water
SD-HD-201B	Moisture Separator/Reheater
SD-MP-201	Main Generator
SD-MP-202	Main Exciter
SD-WR-201	RBCCW System
SD-WT-201	TBCCW System

Status of Preoperational Test/System Demonstration Deficiencies for Those Tests Required by Fuel Load as of July 28, 1983 (Categorization is by Licensee):

Total Deficiencies: 1182
Total Reviewed That Impact Fuel Load (Category 1): 315
Total Reviewed That Impact Initial Criticality: 416
Total Not Reviewed: 2

Status of System Demonstration Deficiencies for Those Tests for Low Power Operations (Categorization is by Licensee):

Total Deficiencies: 138
Total Reviewed That Impact Initial Criticality (Category 4): 84
Total Reviewed That Impact Turbine Roll (Category 6): 45
Total Reviewed That Impact Fuel Load: 14

On July 21, 1983, the applicant submitted for NRC review and approval a list of 20 preoperational tests and system demonstrations that they wished to defer approval on beyond initial fuel loading. In their submittal the licensee indicated that these tests would continue to be worked on up to fuel loading and that it was anticipated that most of the tests would be completed prior to fuel load. Listed below are the subject tests, their completion status as of July 21, 1983, and their completion status as of August 8, 1983.

Test No.	Test Title	July 21 Completion Status	Current Completion Status
PT-CM-201	Containment Monitoring	42%	46%
PT-CM-202	Post 20CA Containment Monitoring	23%	23%
PT-IN-201	Drywell Pneumatics	73%	73%
PT-LD-201	Leakage Detection	80%	80%
PT-MS-201a	MSIV Leakage Control	79%	79%
PT-MS-201b	MSIVs and MS Instrumentation	82%	82%
PT-MS-201c	ADS and MS Safety/Relief	4%	4%
PT-NR-202	Traversing Incore Probe	0%	0%
PT-OG-201	Off-Gas	47%	47%
PT-PC-201	Primary Containment Integrity	86%	86%
PT-PC-203	Containment Isolation Systems	53%	58%
PT-RI-201	RCIC	80%	80%
PT-RP-202	Remote Shutdown	50%	50%
PT-RR-201	Reactor Recirculation	74%	74%
PT-SI-202	Pipe Vibration Monitoring	91%	91%
PT-VP-202	Post LOCA Hydrogen Recombiners	0%	0%
PT-VP-203	Containment Ventilation	24%	36%
SD-PS-201	Process Sampling	37%	37%
SD-SA-201	Service and Instrument Air	90%	92%
SD-SI-201	Vibration Monitoring	40%	46%

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ENCLOSURE 3

CONSTRUCTION STATUS

General

As of August 8, 1983, the licensee reported the status of construction as 100% complete with the exception as noted below. Three systems remain to be turned over to the Station Operating Department. These are the Turbine Electrohydraulic Control System, the Traversing Incore Probe System, and the Intermediate Range Monitor System.

Specific:

Status of Completion of Major Components:

Concrete installation - 100%
Large bore piping installation - 100%
Small bore piping installation - 100%
Large bore hanger installation - 100%
Small bore hanger installation - 100%
Cable pulls - 30 non-safety related cable pulls remain
Conduit - 100%
Cable terminations - 300 terminations remain

Labor Situation:

There is no foreseen craft availability problems.

Critical Path Items:

The first critical path item for station construction is an ongoing inductive heating stress relief program for the recirculation system piping, attachment welds, and portions of the RHR system. This program is being conducted in response to recent concerns generated by observed cracking in piping systems at operating boiling water reactors. The program is anticipated to last approximately two weeks based on a work schedule of seven days per week, two shifts per day. Because of physical interferences, the program entails the removal and reinstallation of a large number of recirculation system piping supports.

The second critical path item is the de-separation of Units 1 and 2 for secondary containment integrity, radwaste system availability, containment monitoring, and station security considerations. Separation was initially implemented to facilitate Unit 2 construction during Unit 1 operation and to minimize the possibility of spreading contamination from Unit 1 to Unit 2.

The licensee is currently formulating the schedule and procedures for de-separation. With respect to station security, this will involve activation of 9 controlled doors, deactivating 8 doors, installation of 3 barriers, and removal of 2 barriers. This is expected to require approximately 200 man-hours of effort over a 7 day period.

The third critical path item is cable tray reinforcement. This program was necessitated by a reanalysis of cable tray loading including containment - imposed loads and involves the installation of tray hanger stiffeners. The program is approximately 60% complete and is expected to last an additional five weeks.

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ENCLOSURE 4

STATUS OF INSPECTION REQUIRED BY MC 2512, 2513, AND 2514

All modules in the NRC Region III inspection program required to be completed prior to fuel load have been completed except as noted below:

MC 2512

Module #	Title	Percent Complete	Reason Not Completed
51056	Review of QA Records on Electrical Systems	0	Note 1
51061	Review of Implementation of QA Procedures on Cables and Terminations	90	Note 1
51064	Observation of Cable and Termination Work	80	Note 1
51065	Review of QA Records on Cables and Terminations	60	Note 1
51066	Review of QA Records on Cables and Terminations	0	Note 1
52061	Review of QA Procedures on Instrumentation	90	Note 1
52063	Observation of Work on Instrumentation Cables and Terminations	80	Note 1
52064	Observation of Work on Instrumentation Cables and Terminations	60	Note 1
52065	Review of QA Records on Instrumentation Cables and Terminations	50	Note 1
52066	Review of QA Records on Instrumentation Cables and Terminations	0	Note 1
64051	Procedures/Fire Prevention/Protection	0	Note 1
64053	Fire Loop Installation	0	Note 1

It is estimated that the MC 2512 program is approximately 88% complete at this time.

Note 1: Inspection activity is scheduled or in progress.

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MC 2513

Module #	Title	Percent Complete	Reason Not Completed
70303	Preoperational Testing Procedure Review	88	Note 1
70304	Preoperational Testing Procedure Review-ESF	83	Note 1
70305	Preoperational Testing Procedure Review-RPS	22	Note 1
70306	Preoperational Testing Procedure Review-Loss of Offsite Power	50	Note 1
70311	Preoperational Testing Procedure Verification	27	Note 1
70312	Preoperational Testing Procedure Witnessing	60	Note 2
70315	Witness ESF	57	Note 2
70317	Witness Reactor Protection System Test	0	Note 2
70316	Witness Emergency Power Redundancy Test	0	Test Not Performed
70320	Evaluation of Test Results	0	Note 3
70322	Evaluation of Test Results-ESF	14	Note 3
70325	Evaluation of Test Results-RPS	0	Note 3
70326	Evaluate Results of Emergency Power Redundancy Test	0	Test Not Performed
70329	Verification That Tests Are Evaluated	11	Note 3
36301	Operational Staffing Inspection	100	
37301	Comparison of As-Built Plant to FSAR Description	20	Note 4
41301	Inspection of Operating Staff Training	0	Note 4
64703	Fire Protection/Prevention	0	Note 5
70370	Testing of Pipe Support and Restraint Systems	0	Note 5
81100	Physical Inspection	90	Note 6
84330	Radioactive Waste Systems Preoperational Liquid and Solid Waste Systems	0	Note 6
84331	Radioactive Waste Systems Preoperational Gas Waste Systems	0	Note 6
94300	Inspection Preparatory to Operating License Issuance	60	Status of Facility

Note 1: Procedure reviews are being conducted concurrently with test witnessing. Testing is still in progress (Enclosure 2).

Note 2: Preoperational testing is still in progress and is being witnessed in accordance with an established schedule.

Note 3: Test results are being evaluated as they become available.

Note 4: To be completed by August 25, 1983.

Note 5: Insufficient resources to date. This inspection will be completed prior to fuel load.

Note 6: Inspection is 90% complete based on Unit 1 inspection. Only tie-ins to Unit 2 remain.

It is estimated that the MC 2513 inspection requirements are approximately 65% complete at this time.

MC 2514

Module #	Title	Percent Complete	Reason Not Completed
35741	QA Program-Audits		
35742	QA Program-Document Control	90	Ongoing
35748	QA Program-Records	90	Ongoing
35749	QA Program-Tests and Experiments	0	
70308	Procedures Review-Integrated Hot Functional Test	0	Note 1
72500	Initial Fuel Loading Procedure Review	0	Note 1
72502	Approach to Critical Procedures Review	0	Note 1
72504	Heatup Phase Procedural Review	0	Note 1
72508	Power Ascension Procedure Review	0	Note 1
72509	Power Ascension Procedure Review-CRD	0	Note 1
72510	Power Ascension Procedure Review-RV or MSIV)	0	Note 1
72512	Power Ascension Procedure Review (RCIC or Recirc Pump Trip)	0	Note 1
72514	Power Ascension Procedure Review (Turbine Trip or Generator Trip)	0	Note 1
72516	Power Ascension Procedure Review Shutdown Outside the Control Room	0	Note 1
72517	Power Ascension Procedure Review (LOP)	0	Note 1
72518	Power Ascension Procedure Review (Core Performance)	0	Note 1
72524	Witness Initial Fuel Loading	0	Fuel Load Has Not Commenced

Note 1: Procedures have yet to be completed.

It is estimated that MC 2514 inspection requirements are approximately 37% complete at this time.

ENCLOSURE 5

W. G. GULDEMOND MEMORANDUM TO
R. D. WALKER DATED AUGUST 5, 1983