Consumers Power Company
Big Rock Point Plant
Docket Number 50-155

CURRENT TECHNICAL SPECIFICATION PAGES

December 15, 1994

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6.7.1 (Continued)

b. The safety limit violation shall be reported within 1 hour to the Commission in accordance with 10 CFR 50.36, as well as to the Vice President - Nuclear Operations and to the NPAD.

c. A report shall be prepared in accordance with 10 CFR 50.36 and 6.9 of this specification. (The safety limit violation and the report shall be reviewed by the PRC.)

d. The report shall be submitted within 14 days to the Commission (in accordance with the requirements of 10 CFR 50.36), to the Vice President - Nuclear Operations and to the NPAD.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained for all structures, systems, components and safety actions defined in the Big Rock Point Quality List. These procedures shall meet or exceed the requirements of ANSI N18.7, as endorsed by CPC-2A.

6.8.2 PRC is responsible for the review of each procedure of 6.8.1 above, and changes thereto (except for Security Implementing Procedures which are reviewed and approved in accordance with the Site Security Plan). The Plant Manager shall approve such procedures and changes prior to implementation.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

a. The intent of the original procedure is not altered.

b. The change is approved by two members (or designated alternates) of the PRC, at least one of whom holds a Senior Resident Operator's License.

c. The change is documented, reviewed by the PRC at the next regularly scheduled meeting and approved by the Plant Manager.

EMERGENCY CORE COOLING SYSTEMS

REACTOR DEPRESSURIZATION SYSTEM

LIMITING CONDITION FOR OPERATION

11.3.1.5
The Reactor Depressurization System (RDS) shall be OPERABLE with:

a. Four RDS valve trains including pilot valves.

b. Four input and output channels including instrumentation given in Table 11.3.1.5

c. Four Uninterruptible Power Supplies (UPS) as described in Specification 11.3.5.3.

d. All mechanical snubbers in service.

APPLICABILITY: POWER OPERATION

ACTION:

a. Should one RDS valve train, one input channel, one output channel, or UPS Power Supply become inoperable in the closed position, the reactor may remain in POWER OPERATION for a period not to exceed seven (7) days, providing the actuating circuitry for the remaining channels is demonstrated to be OPERABLE within 4 hours and at least once each 72 hours until the components are restored to OPERABLE status. If these components are not returned to OPERABLE status within seven (7) days, a normal orderly shut down shall be initiated within one (1) hour and the reactor shall be SHUTDOWN as described in Section 1.2.5(a) within twelve (12) hours and SHUTDOWN as described in Section 1.2.5(a) and (b) within the following 24 hours.

b. Should two or more RDS valve trains including input channel, output channel or UPS Power Supply become inoperable the plant shall be brought to the SHUTDOWN condition within 12 hours and to the COLD SHUTDOWN condition within the following 24 hours.

c. Should one isolation valve or depressurizing valve become inoperable in the open position the plant shall be brought to the COLD SHUTDOWN condition within 12 hours.

d. If the RDS is declared inoperable because of a snubber defect and is not returned to an OPERABLE status within 72 hours, the plant shall be brought in a normal and orderly manner to a COLD SHUTDOWN condition within 12 hours and be maintained in COLD SHUTDOWN until RDS can be declared OPERABLE. If the plant is already in a COLD SHUTDOWN condition, it shall not be started up until all snubbers are OPERABLE.

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PROPOSED TECHNICAL SPECIFICATIONS CHANGE PAGES

December 15, 1994

2 Pages

6.7.1 (Continued)

b. The safety limit violation shall be reported within 1 hour to the Commission in accordance with 10 CFR 50.36, as well as to the Vice President - Nuclear Operations and to the NPAD.

c. A report shall be prepared in accordance with 10 CFR 50.36 and 6.9 of this specification. (The safety limit violation and the report shall be reviewed by the PRC.)

d. The report shall be submitted within 14 days to the Commission (in accordance with the requirements of 10 CFR 50.36), to the Vice President - Nuclear Operations and to the NPAD.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained for all structures, systems, components and safety actions defined in the Big Rock Point Quality List. These procedures shall meet or exceed the requirements of ANSI N18.7, as endorsed by CPC-2A.

6.8.2 PRC is responsible for the review of each procedure of 6.8.1 above, and changes thereto (except for Security Implementing Procedures which are reviewed and approved in accordance with the Site Security Plan). The Plant Manager shall approve such procedures and changes prior to implementation.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

a. The intent of the original procedure is not altered.

b. The change is approved by two members (or designated alternates) of the PRC, at least one of whom holds a Senior Reactor Operator's License.

c. The change is documented, reviewed by the PRC at the next regularly scheduled meeting and approved by the Plant Manager.

EMERGENCY CORE COOLING SYSTEMS

REACTOR DEPRESSURIZATION SYSTEM

LIMITING CONDITION FOR OPERATION

11.3.1.5 The Reactor Depressurization System (RDS) shall be OPERABLE with:

a. Four RDS valve trains including pilot valves.

b. Four input and output channels including instrumentation given in Table 11.3.1.5.

c. Four Uninterruptible Power Supplies (UPS) as described in Specification 11.3.5.3.

d. All mechanical snubbers in service.

APPLICABILITY: POWER OPERATION

ACTION:

a. Should one RDS valve train, one input channel, one output channel, or UPS Power Supply become inoperable in the closed position, the reactor may remain in POWER OPERATION for a period not to exceed seven (7) days. If these components are not returned to OPERABLE status within seven (7) days, a normal orderly shut down shall be initiated within one (1) hour and the reactor shall be SHUTDOWN as described in Section 1.2.5(a) within twelve (12) hours and SHUTDOWN as described in Section 1.2.5(a) and (b) within the following 24 hours.

b. Should two or more RDS valve trains including input channel, output channel or UPS Power Supply become inoperable the plant shall be brought to the SHUTDOWN condition within 12 hours and to the COLD SHUTDOWN condition within the following 24 hours.

c. Should one isolation valve or depressurizing valve become inoperable in the open position the plant shall be brought to the COLD SHUTDOWN condition within 12 hours.

d. If the RDS is declared inoperable because of a snubber defect and is not returned to an OPERABLE status within 72 hours, the plant shall be brought in a normal and orderly manner to a COLD SHUTDOWN condition within 12 hours and be maintained in COLD SHUTDOWN until RDS can be declared OPERABLE. If the plant is already in a COLD SHUTDOWN condition, it shall not be started up until all snubbers are OPERABLE.