AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE December 10, 1982
COMPLETED BY T. P. Matthews
TELEPHONE (402)536-4733

MONTH November, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	301.8	17	376.3
2	303.4	18	376.1
3	306.0	19	376.3
4	305.4	20	376.0
5	304.8	21	374.7
6	306.3	22	374.9
7	306.7	23	374.8
8	306.5	24	375.7
9	308.5	25	378.4
10	307.8	26	378.5
11	316.9	27	378.6
12	344.0	28	379.5
13	369.0	29	379.5
14	374.9	30	378.9
15	375.9	31	
16	376.5		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-285
DATE December 10, 1982
COMPLETED BY T. P. Matthews
TELEPHONE (402)536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: November, 1982
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	8,016.0	80,497.0
12. Number Of Hours Reactor Was Critical	720.0	7,801.5	64,040.5
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	720.0	7,787.5	62,877.5
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	788,152.6	10,832,932.7	77,534,762.8
17. Gross Electrical Energy Generated (MWH)	267,634.0	3,633,373.9	25,707,319.5
18. Net Electrical Energy Generated (MWH)	251,817.1	3,455,732.6	24,303,601.1
19. Unit Service Factor	100.0	97.1	78.1
20. Unit Availability Factor	100.0	97.1	78.1
21. Unit Capacity Factor (Using MDC Net)	73.2	90.2	65.6
22. Unit Capacity Factor (Using DER Net)	73.2	90.2	65.3
23. Unit Forced Outage Rate	0.0	2.9	3.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
1983 refueling outage commenced December 7, 1982, earlier than planned, due to a forced outage resulting from turbine bearing and rotor blade problems.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: March 15, 1983
26. Units In Test Status (Prior to Commercial Operation): N/A

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH November, 1982

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE December 10, 1982
COMPLETED BY T. P. Matthews
TELEPHONE (402)536-4733

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns during the month of November, 1982.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending November 1982 .

1. Scheduled date for next refueling shutdown. — March, 1984

2. Scheduled date for restart following refueling. — May, 1984

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be?

   A Technical Specification Change

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. —

   c. If no such review has taken place, when is it scheduled? —

4. Scheduled date(s) for submitting proposed licensing action and support information. — February, 1984

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

6. The number of fuel assemblies:
   a) in the core — 133 assemblies
   b) in the spent fuel pool — 237 "
   c) spent fuel pool storage capacity — 483 "
   d) planned spent fuel pool storage capacity — 728 "

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1985

Prepared by J K Gasper Date December 1, 1982

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

November, 1982
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station began increasing power November 11, 1982 from 65% power while Nebraska City went off line. The plant was stabilized at 78% power November 13, 1982 and remained at that power level through the rest of the month.

Four bundles of new fuel were received November 3, 1982.

Annual licensed operator requalification examinations are under way and will be completed December 2, 1982.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

LER Number	Deficiency
NONE	

B. CHANGES IN OPERATING METHODS

NONE

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which resulted in Operation Incidents and are not reported elsewhere in the report:

Operation Incidents		Deficiency
OI-1597	ST-SUTEMP-1	During performance of Surveillance Test, TIA-135 and TIA-136 indicators were erroneously logged.
OI-1601	ST-ESF-6	DG-1, governor booster pump motor was removed and sent out for repair without maintenance order.
OI-1602	ST-DG-2, F.1, F.2, F.3	During performance of ST-DG-2, several switches noticed to be out of calibration tolerance. Speed Sensing Switch, jacket water pressure, lube oil high temperature alarm, fuel oil base tank level.

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS (Continued)

Operation Incidents		Deficiency
OI-1606	ST-CSA-1, F.6	While performing test, it was discovered that Group 4 PPDIL & PDIL setpoints set a higher value than they were supposed to.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure		Description
ST-FE-1		Visual inspection of spent fuel. Completed per procedure. This procedure did not constitute an unreviewed safety question per 10CFR50.59 since any potential accidents have been evaluated in the FSAR section on "Final Handling Accidents".
SP-SFS-1		Shipment of Spent Fuel in NL1-1/2 Cask. Completed per procedure. This procedure did not constitute an unreviewed safety question per 10CFR50.59 since redundant rigging was used and a potential fuel handling accident has been analyzed in the FSAR.
SP-UF_6-1	9-23-82 9-27-82 9-29-82 10-1-82 10-5-82	Uranium Hexafluoride Storage Cylinder External Visual Inspection. Completed per procedure. Performance of this procedure did not constitute an unreviewed safety question as defined by 10CFR50.59. Handling and storage of the UF_6 is in accordance with license SMC-1420 and thus does not affect the station operating license. The safety impact of UF_6 release would be less severe than that of other types of chemical releases which have already been evaluated.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure		Description
SP-UF_6-2	9-23-82 9-27-82 9-29-82 10-1-82 10-5-82	Off-Loading of UF_6 Cylinders. Completed per procedure. Performance of this procedure did not constitute an unreviewed safety question as defined by 10CFR50.59. Handling and storage of the UF_6 is in accordance with license SMC-1420 and thus does not affect the station operating license. The safety impact of UF_6 release would be less severe than that of other types of chemical releases which have already been evaluated.
SP-NFR-1		Fuel Receipt Procedure. Completed per procedure. Performance of SP-NFR-1 did not constitute an unreviewed safety question per 10CFR50.59. New fuel receipts were performed in strict accordance with the station operating license and applicable regulations. No possibility existed of a significant radiological release in the event of mishandling.
SP-FAUD-1		Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.

E. RESULTS OF LEAK RATE TESTS

NONE

F. CHANGES IN PLANT OPERATING STAFF

NONE

G. TRAINING

Training for November, 1982:

Operators, NRC licensed personnel were administered an annual requalification exam.

Non-licensed operators were trained on systems, theory and procedures on an as needed basis.

All operators received lectures on Emergency Procedures, the newly installed H_2 analyzer SER, and reactor theory.

All plant personnel attended general employee training refresher on a scheduled basis in preparation for the 1983 outage.

Maintenance received training on systems as scheduled.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

NONE

II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
16875	10-11-82	RMO/60 pump will not start breaker tripped free.	Removed motor and replaced.
16877	10-20-82	Charging Pump, CH-1A packing cooling tank overfills when pump is shutdown.	Completed per MP-CH-1-1.
16913	10-25-82	HCV-2633 valve has no indication light.	Valve replaced.
16889	10-20-82	B/PIC-905 Sigma failed to reset.	Removed, repaired and reinstalled sigma.
16797	10-28-82	Replace motor & brake on CRDM-17.	Replaced rectifier.
16937	10-29-82	NI "A" wide range channel irratic indication.	Re-calibrated.
16936	10-29-82	NI "D" wide range channel B-10 detector irratic indication.	Re-calibrated.
16921	10-28-82	S/G Pressure sigma low pressure trip light blinks.	Replaced indicating light.
16961	10-28-82	ROD #23 will not operate.	Repaired NC contract in raise relay.
17208	11-5-82	TIA-221 sigma appears to have failed.	Replaced alarm circuit.
16990	11-22-82	RM-055A needs to be cleaned.	Cleaned RM-055.
16935	10-28-82	RC-2B Steam Generator leak on handhole.	Machined gasket surface and repaired.
16933	10-28-82	RC-2A Steam Generator south handhole leaks.	Machined gasket surface and repaired.
16934	10-27-82	RC-2A Steam Generator handhole leaks.	Machined gasket surface and repaired.
16798	10-27-82	RC-2B support rod broken.	Repair per procedure.

W. G. Gates
W. G. Gates
Plant Manager

Omaha Public Power District

1623 HARNEY ■ OMAHA, NEBRASKA 68102 ■ TELEPHONE 536-4000 AREA CODE 402

December 13, 1982
LIC-82-403

Mr. Richard C. DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Reference: Docket No. 50-285

Dear Mr. DeYoung:

Please find enclosed ten (10) copies of the November Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,

W. C. Jones
Division Manager
Production Operations

WCJ/TLP:jmm

Enclosures

cc: NRC Regional Office
Office of Management & Program Analysis (2)
Mr. R. R. Mills - Combustion Engineering
Mr. T. F. Polk - Westinghouse
Mr. L. A. Yandell - NRC Senior Resident Inspector
Nuclear Safety Analysis Center
NRC File