NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION APPROVED BY OME (12-81) 10 CFR 50 LICENSEE EVENT REPORT 3150-0011 CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 
 P
 A
 S
 E
 S
 1
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
 0
0 1 CON'T REPORT L 60 5 0 0 0 3 8 7 0 1 2 3 0 8 3 6 0 1 2 7 8 4 9 0 1 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) 0 2 | During the Unit 1-Unit 2 tie-in outage, one of the 'B' Reactor Protection System 0 3 [Motor Generator Set Electrical Protection Assembly (EPA) breakers tripped and 0 4 caused a primary containment isolation resulting in a loss of shutdown (S/D) cool-0 5 ing due to the closure of S/D cooling suction inboard and outboard isolation 0 6 valves. There were no effects to public health and safety due to the Unit's compliance with ACTION Statement b of Tech. Spec. 3.9.11.1. 0 7 0 8 80 SYSTEM CAUSE CAUSE COMP VALVE CODE COMPONENT CODE SUBCODE B E (12) A 15 (11) A (13) KTBRK 4 Z 16 0 9 C 19 SEQUENTIAL REPORT NO. CODE REPORT REVISION NO. 17 REPORT 0 3 0 28 32 HOURS 22 ATTACHMENT NPRD-4 SUBMITTED FORM SUB. ACTION FUTURE TAKEN ACTION EFFECT ON PLANT METHOD COMPONENT PRIME COMP. MANUFACTURER 26 SUPPLIER Z 2 N 24 Z 20 0 0 0 0 G 0 8 0 A 18 X 19 A 25 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) 10 The loss of shutdown (S/D) cooling was caused by the trip of the EPA breaker and 1 subsequent PCIS actuation. Reactor coolant circulation was estd. through the Fuel [Pool Cooling system. (Core Spray system also available.) Rx. coolant temp. is 13 monitored once/hr. in Mode 5 per plant procedures. Cause of brkr. trip is under 1 4 investigation. Failed brkr. was replaced and S/D cooling re-estd. within 3 hours. FACILITY METHOD OF OTHER STATUS (30) DISCOVERY DESCRIPTION (32) 1 5 H 28 0 0 0 29 NA A [3] Operator Observation ACTIVITY CONTENT 12 13 45 80 (35) RELEASED AMOUNT OF ACTIVITY LOCATION OF RELEASE (36) 6 Z 33 Z 34 NA NA 1 10 PERSONNEL EXPOSURES 11 60 NUMBER NA 1.5 12 PERSONNEL INJURIES DESCRIPTION (41) NUMBER 180000 NA 8402080242 840127 1 11 12 LOSS OF OR DAMAGE TO PACILITY TYPE DESCRIPTION 80 PDR ADOCK 05000387 Z (42) PDR NA 1 9 DESCRIPTION (45) ISSUED NRC USE ONLY 20 N 44 NA 111111 NAME OF PREPARER L.A. KUCZYNSKI PHONE (717)542-2181 Ext. 3759



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

January 27, 1984

Dr. Thomas E. Murley Regional Administrator, Region I U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, PA 19406

SUSQUEHANNA STEAM ELECTRIC STATION LICENSEE EVENT REPORT 83-172/03L-0 ER 100450 FILE 841-23 PLA- 2058

Docket No. 50-387 License No. NPF-14

F62.

Dear Dr. Murley:

Attached is Licensee Event Report No. 83-172/03L-0. This event was determined to be reportable per Technical Specification 6.9.1.9.b, in that with the Unit in Mode 5 (Refuel), a primary containment isolation occurred when a 'B' Reactor Protection System Motor Generator Set Electrical Protection Assembly (EPA) breaker tripped. The containment isolation resulted in the loss of shutdown cooling. Alternate reactor coolant circulation was established through the Fuel Pool Cooling System and reactor coolant temperature was already being monitored once per hour per plant procedure. The Core Spray System was also available to provide coolant circulation, if required. The EPA breaker was replaced and shutdown cooling re-established within three hours.

A Keiser

H.W. Keiser Superintendent of Plant-Susquehanna

LAK/pjg

cc: L.R. Plisco
Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 52
Shickshinny, PA 18655

Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555