#### AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	Fort Calhoun Station
DATE	August 10, 1983
COMPLETED BY	T. P. Matthews
TELEPHONE	(402)536-4733

MONTH	July, 1983
DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	448.5
2	446.7
3	443.7
4	444.5
5	445.9
6	447.2
7	446.7
8	446.3
9	445.9
10	444.7
11	444.0
12	444.1
13	444.4
14	444.0
15	443.0
16	443.5

AVERAGE DAILY POWER LEVEL (MWe-Net)
443.4
441.8
440.3
439.2
438.5
438.4
438.1
439.4
443.0
443.3
442.3
441.9
443.1
442.7
442.2

## INSTRUCTIONS

8308190266 830810 PDR ADOCK 05000285 R

PDR

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On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

#### **OPERATING DATA REPORT**

DOCKET NO.	50-285
DATE	August 10, 1983
COMPLETED BY	T. P. Matthews
TELEPHONE	(402)536-4733

## **OPERATING STATUS**

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I. Unit Name: Fort Calho	oun Station	Notes
2. Reporting Period:July,	1983	
3. Licensed Thermal Power (MWt):	1500	
4. Nameplate Rating (Gross MWe):	501	
5. Design Electrical Rating (Net MW	e):478	
6. Maximum Dependable Capacity (	Gross MWe): 501	
7. Maximum Dependable Capacity (	Net Mike): 478	

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: N/A

# 9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_N/A

10. Reasons For Restrictions, If Any: \_\_\_\_\_None

	This Month	Yrto-Date	Cumulative
11. Hours In Reporting Period	744.0	5,088.0	86,329.0
12. Number Of Hours Reactor Was Critical	744.0	2,830.9	66,220.9
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	744.0	2,732.0	65,729.5
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,110,048.8	3,727,158.6	81,343,707.0
17 Gross Electrical Energy Generated (MWH)	346,692.0	1,172,852.0	26,596,331.7
18. Net Electrical Energy Generated (MWH)	329,773.5	1,111,631.3	25,441,665.7
19. Unit Service Factor	100.0	53.7	76.1
20. Unit Availability Factor	100.0	53.7	76.1
21. Unit Capacity Factor (Using MDC Net)	92.7	45.7	63.9
22. Unit Capacity Factor (Using DER Net)	92.7	45.7	62.0
23. Unit Forced Outage Rate	0.0	1.8	3.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

COMMERCIAL OPERATION

25. If Shut Down At End Of Report Period, Estimated Date	e of Startup:		
26. Units In Test Status (Prior to Commercial Operation):	N/A	Forecast	Achieved
INITIAL CRITICALITY			
INITIAL ELECTRICITY			

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# UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. UNIT NAME PATE COMPLETED BY TELEPHONE DOCKET NO. 50-285 Fort Calhoun Station August 10, 1983 T. P. Matthews (402)536-4733

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REPORT MONTH \_ July, 1983

No.	Date	Type <sup>1</sup> Duration (Hours)	Reason <sup>2</sup> Method of	Shutting Shutting Free Free Free Boon	System ass Code 4m	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
							There were no unit shutdowns for the month of July, 1983.
1 F Fe S: Scl	rced heduled	2 Reason: 3-Equipmen B-Maintenan C-Refueling D-Regulator E-Operator 1 F-Administra G-Operation H-Other (Ex	t Failure (Explained of Test Restriction Training & Licen trive al Error (Explain plain)	in) se Examination 1)	3 Meths 1-Mar 2-Mar 3-Aut 4-Oth	d: uual uual Scram. omatic Scram. er (Explain)	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit 1 - Same Source

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Refueling Information Fort Calhoun - Unit No. 1

Report for the month ending July 1983

- Scheduled date for next refueling shutdown. 1. 2. Scheduled date for restart following refueling. 3. Will refueling or resumption of operation thereafter require a technical specification change or other
  - a. If answer is yes, what, in general, will these be?

license amendment?

A Technical Specification Change

- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.
- c. If no such review has taken place, when is it scheduled?
- 4. Scheduled date(s) for submitting proposed licensing action and support information.
- 5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

May	1984	

Yes

Methodology - October 1983 Tech. Specs. - February 1984

6.	The number of	ruel assemblies:	a) in the c	core	133	assemblies
			b) in the s	spent fuel pool	265	
			c) spent fu storad	uel pool je capacity	483	"
			d) planned storad	spent fuel pool je capacity	728	
7.	The projected discharged to	date of the last the spent fuel p	refueling to	that can be the present		

licensed capacity.

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Date August 1, 1983

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## OMAHA PUBLIC POWER DISTRICT Fort Calhoun Station Unit No. 1

July, '983 Monthly Operations Report

### . OPERATIONS SUMMARY

Fort Calhoun Station operated at nominal 100% power during the month of July. Repairs were completed on condensate pump FW-2C. The preparation of hot license candidates for the S.R.O. examination early in August continued during July. The staff at Fort Calhoun Station participated in two meetings with the NRC dealing with health physics/radiochemistry and a proposed Technical Specification on the Reactor Protective System.

No safety valve or PORV challenges occurred.

# A. PERFORMANCE CHARACTERISTICS

None

B. CHANGES IN OPERATING METHODS

None

#### C. RESULTS OF SURVEILL/ ICE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which resulted in Operation Incidents and are not reported elsewhere in the report:

Operation Incident

## Deficiency

0I-1731 ST-RPS-12, F.2

During routine plant operation at approximately 100% power, the "D" RFS Channel ASI internal tilt setpoint was found to have drifted out of specifications.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

SP-EGT-1

Testing of Heater SA Outlet Heat Exchanger Tubes

This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved testing of non-safety related equipment. Monthly Operations Report = July, 1983 Page Two

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (continued)

System Acceptance Committee Packages for July, 1983:

## Package Description/Analysis

DCR 76-88 Heater Drain/Moisture Separator Drain Valve.

This modification changed out the valve internals to improve flow characteristics. This modification has no adverse effect on the safety analysis.

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EEAR FC-80-46 Emergency Feedwater Storage Tank Backup Fill.

This modification provides redundant water supply for the emergency feedwater storage tank. This modification increases plant safety and consequently has no adverse effect on the safety analysis.

EEAR FC-80-87 "D" Inverter Output Breaker.

This modification replaced a faulty breaker with a similar breaker of comparable quality. This modification has no adverse effect on the safety analysis.

EEAR FC-80-72 Immersion Heater Transformer for D1.

This modification replaced a defective transformer. This modification has no adverse effect on the safety analysis.

# E. RESULTS OF LEAK RATE TESTS

Measured leakage per surveillance test ST-CONT-2, F.2 (Personnel Air Lock Leak Rate Test) was 3100 sccm. This leakage does contribute towards the .6 La Technical Specification limit. Leakage to date (cumulative) is approximately 56,186.3 sccm which is well below the 62,951 sccm limit.

F. CHANGES IN PLANT OPERATING STAFF

None

# G. TRAINING

General employee training was conducted as scheduled. S.R.O. upgrade training was conducted for six people who are to sit for N.R.C. examinations in August. Training included lectures and walkthroughs. New monitor team members were given initial training on the emergency plan and emergency plan implementing procedures.

- Monthly Operations Report July, 1983 Page Three
  - H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59.

None

# II. MAINTENANCE (Significant Safety Related)

M.O. #	DATE	DESCRIPTION	CORRECTIVE ACTION
20282	6/2/83	CH-1B has primary leak into packing coolant tank	Installed new packing
19413	3/23/83	HCV-1385 replace grease	Complete
20168	5/27/83	AC-3A disassemble motor and inspect for cause of increased vibration	Tightened bearings
19750	3/31/83	CEA 6 and 12 malfunctioned	Repaired wire
21015	7/11/83	APD trip setpoint drifted	Replaced dual amp module bistable
20517	5/31/83	"A" control channel deviation light on	Adjusted
20908	7/1/83	RC-133 pressurizer mini-spray blowing around packing	Completed per PRC approved procedure
20894	7/8/83	AI-31B linear power channel "B" negative APD limit drifting causing APD channel trip	Replaced F4,5 module and reper- formed applicable sections of ST-RPS-12 F.2
21089	7/20/83	RM-050/51 pump discharge filter leaks - gasket cracked	Replaced gasket
18888	3/3/83	Replace studs on RC-3B	Completed per SP-RC-11
18881	3/1/83	HCV-327 valve seat and disc machining	Completed per procedure
18483	2/9/83	RPS Channel A 112/H loop has bad power supply loop pegged high	Replaced filter capacitor, checked loop per CP-A/112H
19419	3/25/83	HCV-2974 replace grease	Complete
19418	3/25/83	HCV-2954 replace grease	Complete
19416	3/25/83	HCV-2914 replace grease	Complete
19417	3/25/83	HCV-2934 replace grease	Complete
19412	3/25/83	HCV-1386 replace grease	Complete

# Monthly Operations Report July, 1983 Page Four

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# II. MAINTENANCE (continued)

M.O. #	DATE	DESCRIPTION	CORRECTIVE ACTION
19613	3-29-83	VA-3A 480V Breaker trip device	Installed new trip device
19603	3/28/83	M-1-2,3,4	Inspect M-contactors per NRC Bulletin
19602	3/28/83	RPS circuit breaker inspection	Check RPS circuit breakers
18176	2/25/83	HCV-315 SI loop valve	Regreased and checked
18174	2/25/83	HCV-312 SI loop valve	Regreased and checked
19916	4/13/83	CH-1B charging pump	Repacked
19262	3/19/83	Inspect the gearcase grease in Limitorque actuators HCV-218-3, 347, 308	Completed per procedure
19260	3/18/83	Inspect the gearcase grease in Limitorque actuators HCV-311, 312, 314, 315, 317, 318, 321, 320, 329, 327, 331, 333, 150, 151	Completed per procedure
18116	3/7/83	CEDM #25 leaks	Repaired per MP-RC-10-4-A and MP-RC-10-4-B
18117	3/7/83	CEDM #7 leaks	Repaired per MP-RC-10-4-C
18118	3/7/83	CEDM #13 leaks	Repaired per MP-RC-10-4-C
18948	3/13/83	RC-3A removal and replacement of 3 RC pump studs	Completed per MP-RC-3-7
19481	3/28/83	Replace the alarm relay RL2 on board VO 72 in inverter B & C	Replaced
19420	3/22/83	HCV-347 replace grease	Complete
18784	3/7/83	HCV-327 SI loop isolation valve remove motor and repack grease in gear box	Complete
18785	3/7/83	HCV-329 SI loop isolation valve remove motor and repack grease in gear box	Complete
18786	3/7/83	HCV-331 SI loop isolation valve remove motor and repack grease in gear box	Complete
19357	3/25/83	HCV-1042C replace grease	Replaced
19356	3/23/83	HCV-1041C replace grease	Replaced

. . Monthly Operations Report July, 1983 Page Five

II. MAINTENANCE (continued)

M.O. #	DATE	DESCRIPTION	CORRECTIVE ACTION
19407	4/11/83	Diesel generator engine driven air compressor unloading air is leaking	Replaced air compressor
19559	3/28/83	FW-1A condenser tube leaking circulating water into hotwell	Plugged tubes per procedure
19582	3/28/83	Sequence timer for FW-6 did not work	Tightened loose part on internal contact
19342	3/19/83	LCV-218-3 change gear grease	Complete
19314	3/25/83	HCV-258 will not go fully open	Greased and operated correctly
19291	3/23/83	HCV-383-3 change grease	Complete
19278	3/19/83	HCV-150, 151 add grease	Complete
19276	3/19/83	HCV-311, 314, 317, 320 add grease	Complete
19275	3/19/83	HCV-2914, 2934, 2954 add grease	Complete
19265	3/17/83	Repair reactor vessel stud hole #21	Completed per procedure
19264	3/16/83	HCV-383-3 and 383-4 inspect grease	Replaced grease
18787	3/7/83	HCV-333 SI loop isolation valve remove motor and repack grease in gear box	Complete

N. M. Mates

W. G. Gates Manager Fort Calhoun Station

Omaha Public Power District 1623 Harney Omaha. Nebraska 68102 402/536-4000

> August 12, 1983 LIC-83-193

Mr. Richard C. DeYoung, Director Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission Washington, D.C. 20555

Reference: Docket No. 50-285

Dear Mr. DeYoung:

Please find enclosed ten (10) copies of the July Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,

W. C. Jones Division Manager Production Operations

WCJ/TPM:jmm

Enclosures

cc: NRC Regional Office Office of Management & Program Analysis (2) Mr. R. R. Mills - Combustion Engineering Mr. T. F. Polk - Westinghouse Nuclear Safety Analysis Center INPO Records Center NRC File

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