

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE August 15, 1983
 COMPLETED BY C. W. Smyth
 TELEPHONE (717) 948-8551

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit 1
2. Reporting Period: July, 1983
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744.	5087.	78120.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	839.5
14. Hours Generator On-Line	0.0	0.0	31180.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	0.0	76531071.
17. Gross Electrical Energy Generated (MWH)	0.	0.	25484530.
18. Net Electrical Energy Generated (MWH)	0.	0.	23840053.
19. Unit Service Factor	0.0	0.0	39.9
20. Unit Availability Factor	0.0	0.0	39.9
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	39.0
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	37.3
23. Unit Forced Outage Rate	100.0	100.0	56.0
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-1
 DATE August 15, 1983
 COMPLETED BY C. W. Smyth
 TELEPHONE (717) 948-8551

MONTH July, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

50-289

DOCKET NO.

UNIT NAME

August 15, 1983

DATE

C. W. Smyth

COMPLETED BY

(717) 948-8551

TELEPHONE

REPORT MONTH July, 1983

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	83/07/01	F	744	D	I	N/A	ZZ	ZZZZZZ	Regulatory Reevaluation Order

1	Forced Scheduled	2	Reason: A-Equipment Failure (Explain) B-Maintenance of Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Administrative G-Operational Error (Explain) H-Other (Explain)	3	Method: 1-Manual 2-Manual Scram, 3-Automatic Scram, 4-Other (Explain)	4	Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NURIG- 0161)
						5	Exhibit I - Same Source

OPERATIONS SUMMARY

The unit was shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat Removal System. With OTSG repairs completed, a hydrogen peroxide flush of the RCS was performed to remove deposits of sulfur contamination.

In conjunction with the flush the following major components/systems have been exercised: reactor coolant pumps, CRDMs, makeup pumps, and pressurizer.

Additionally, most primary support systems were also run during the month and vacuum was drawn in the secondary. Near the end of the month, a comprehensive program of ESAS testing was commenced that lasted into the following month.

To date, all components and systems have run with little or no problems.

MAJOR SAFETY RELATED MAINTENANCE

The following major maintenance occurred during the month:

DH-V14B Repair

DH-V14B was found to be leaking through and was disassembled. The seat ring was replaced and the valve reassembled and tested satisfactorily.

DH-V5A

The magnetic brake on DH-V5A was replaced in order to upgrade the valve for more severe environmental conditions. The valve was then tested satisfactorily.

Local Leak Rate Testing

This program continued with the satisfactory testing of HM-V4A and LR-V49.

Valve Operator Thyrectors Change Out

Thyrectors in 22 of 47 valve operators were replaced. The replacement was necessary to improve the environmental qualification. The remaining thyrectors will be replaced next month.

Purge Valves

Hard stops were installed in AH-V-1B and C to prevent their discs from drifting past their intended seat position. AH-V1C tested satisfactorily. AH-V1B will be tested next month.

Snubber Testing

The snubber testing and repair program continued during the month. No failures were identified.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1987 is the last refueling discharge which allows full core 100% load capacity (177 fuel assemblies).

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE August 15, 1983
 COMPLETED BY C. W. Smyth
 TELEPHONE (717) 948-8351

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit 1
2. Reporting Period: July, 1983
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744.</u>	<u>5087.</u>	<u>78120.</u>
12. Number Of Hours Reactor Was Critical	<u>0.0</u>	<u>0.0</u>	<u>31731.8</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>839.5</u>
14. Hours Generator On-Line	<u>0.0</u>	<u>0.0</u>	<u>31180.9</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0.0</u>	<u>0.0</u>	<u>76531071.</u>
17. Gross Electrical Energy Generated (MWH)	<u>0.</u>	<u>0.</u>	<u>25484330.</u>
18. Net Electrical Energy Generated (MWH)	<u>0.</u>	<u>0.</u>	<u>23840053.</u>
19. Unit Service Factor	<u>0.0</u>	<u>0.0</u>	<u>39.9</u>
20. Unit Availability Factor	<u>0.0</u>	<u>0.0</u>	<u>39.9</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0.0</u>	<u>0.0</u>	<u>39.0</u>
22. Unit Capacity Factor (Using DER Net)	<u>0.0</u>	<u>0.0</u>	<u>37.3</u>
23. Unit Forced Outage Rate	<u>100.0</u>	<u>100.0</u>	<u>56.0</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-1
 DATE August 15, 1983
 COMPLETED BY C. W. Smyth
 TELEPHONE (717) 948-8551

MONTH July, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

50-289

DOCKET NO.

UNIT NAME

August 15, 1983

DATE

C. W. Smyth

COMPLETED BY

TELEPHONE (717) 948-8551

REPORT MONTH July, 1983

No.	Date	Type	Duration (Hours)	Reason	Method of Shutting Down Reactor	Licensee Event Report #	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence
1	83/07/01	F	744	D	I	N/A	ZZ	ZZZZZZ	Regulatory Restraining Order

1	Forced or Scheduled	2	Reason: A-Equipment Failure (Explain) B-Maintenance or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Administrative G-Operational Error (Explain) H-Other (Explain)	3	Method: 1-Manual 2-Manual Scram 3-Automatic Scram 4-Other (Explain)	4	Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (EIR) File (NURLE-0161)
						5	Exhibit I - Same Source

OPERATIONS SUMMARY

The unit was shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat Removal System. With OTSG repairs completed, a hydrogen peroxide flush of the RCS was performed to remove deposits of sulfur contamination.

In conjunction with the flush the following major components/systems have been exercised: reactor coolant pumps, CRDMs, makeup pumps, and pressurizer.

Additionally, most primary support systems were also run during the month and vacuum was drawn in the secondary. Near the end of the month, a comprehensive program of ESAS testing was commenced that lasted into the following month.

To date, all components and systems have run with little or no problems.

MAJOR SAFETY RELATED MAINTENANCE

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Unknown

3. Scheduled date for restart following refueling:

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If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 732. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1987 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).



GPU Nuclear Corporation
Post Office Box 480
Route 441 South
Middletown, Pennsylvania 17057-0191
717 944-7621
TELEX 84-2386
Writer's Direct Dial Number:

August 15, 1983
5211-83-234

Office of Management Information
and Program Control
Attn: W. C. McDonald
c/o Distribution Services Branch
DPC, ADM
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Sir:

Three Mile Island Nuclear Station, Unit 1 (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report for July

Enclosed please find two (2) copies of the July Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

R. D. Hukill
R. D. Hukill
Director, TMI-1

HDH:vjf

Enclosures

cc: V. Steilo
Dr. T. E. Murley

IE74
1/1