

JUL 27 1983

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. STN 50-447

GENERAL ELECTRIC COMPANY

GENERAL ELECTRIC STANDARD SAFETY ANALYSIS REPORT

(GESSAR II BWR/6 NUCLEAR ISLAND DESIGN)

NOTICE OF ISSUANCE OF FINAL DESIGN APPROVAL

Notice is hereby given that the staff of the Nuclear Regulatory Commission (NRC staff) has issued Final Design Approval No. FDA-1 dated July , 1983 for the BWR/6 Nuclear Island design described in General Electric Standard Safety Analysis Report (GESSAR II). GESSAR II was reviewed by the NRC staff pursuant to Appendix O to 10 CFR Part 50.

GESSAR II contains final safety-related design information for the nuclear island portion of a BWR-6/Mark III containment boiling water reactor type nuclear power plant, which includes the nuclear steam supply system (NSSS), engineered safety feature systems, reactor building (including shield building and containment), auxiliary building, control building, radwaste building, fuel handling building, and related systems and structures. The BWR/6 Nuclear Island reference design is for a facility which would operate at a core thermal power level of 3730 megawatts (1269 megawatts electrical, nominal net).

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The Safety Evaluation Report (SER) and Supplement 1 thereto document the results of the NRC staff's review and evaluation of GESSION II, including Amendments 1 through 16 thereto. The SER also addresses the comments of the Advisory Committee on Reactor Safeguards (ACRS) as reflected in its report to the Commission dated June 15, 1983. A copy of the ACRS report is included in Appendix F to SER Supplement 1.

FDA-1 provides NRC staff approval of the final BWR/6 Nuclear Island design described in GESSION II, including Amendments 1 through 16 thereto. By the issuance of FDA-1, the NRC staff has determined that the design is acceptable for referencing in utility applications for operating licenses for those plants that referenced the Preliminary Design Approval for the GESSION-238 Nuclear Island Design (PDA-1) at the construction permit stage with the exception of those features of the design for which the staff has identified requirements that differ from those described in the GESSION II document. These design features relate to fuel rod internal pressure and post accident monitoring instrumentation which are discussed in the SER and incorporated as conditions of FDA-1. These conditions must be satisfactorily resolved prior to the NRC issuing an operating license to a facility referencing the GESSION II design. The BWR/6 Nuclear Island design as described in GESSION II, subject to the conditions of the FDA-1, shall be utilized by and relied upon by the NRC staff and the ACRS in their reviews of facility operating license applications incorporating by reference GESSION .I unless there exists significant new information which substantially affects the determinations in FDA-1 or other good cause.

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Issuance of FDA-1 does not constitute a commitment to issue a permit or license, or in any way affect the authority of the Commission, Atomic Safety and Licensing Appeal Board, Atomic Safety and Licensing Boards and other presiding officers in any proceeding under Subpart G of 10 CFR Part 2. This action only approves the design of a facility for use for reference purposes in applications for operating licenses for nuclear power plants that referenced PDA-1 at the construction permit stage. It does not authorize the operation of any nuclear power plant or any other facility. The environmental impacts associated with any facility proposed to be operated utilizing the approved reference design will be considered in accordance with the Commission's regulations in 10 CFR Part 51.

FDA-1 is effective as of its date of issuance and shall expire on July , 1986 unless extended by the NRC staff. The expiration of FDA-1 on July , 1986, shall not affect its use for reference in operating license applications docketed prior to such date.

A copy of (1) Final Design Approval No. FDA-1 dated July , 1983 and Attachment A thereto; (2) the report of the Advisory Committee on Reactor Safeguards dated June 15, 1983; (3) the NRC staff's Safety Evaluation Report, NUREG-0979, dated April 1983; and Supplement 1 thereto dated July 1983, and (4) the General Electric Standard Safety Analysis Report GESSION II and Amendments 1 through 16 thereto are available for public inspection at the Commission's Public Document Room at 1717 H Street, N.W., Washington, D. C. 20555.

A copy of FDA-1 and Attachment A thereto, may be obtained upon request to

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the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention:
Director, Division of Licensing. Copies of the Safety Evaluation Report
and Supplement I thereto may be purchased at current rates from the National
Technical Information Service, Springfield, Virginia 22161.

Dated at Bethesda, Maryland, this

FOR THE NUCLEAR REGULATORY COMMISSION

(S)
Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Approved by Scientific Council on 7/19/83

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