	EVENT REPORT	APPROVED BY OMB 3150-00:1 EXPIRES 4-30-82
CONTROL BLOCK:	(PLEASE PRINT OR TYPE ALL REQUIRED INFOR	MATION)
0 1 S C N E E 1 2 0 0 - 0 0 0 14 15 LIGENSE N	0 0 - 0 0 3 4 1 1 1 1 4 4 UMBER 25 25 LICENSE TYPE 30	57 GAT 50 (5)
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CATAL GEODINI LIGHT	0	
0 2 On June 23, 1983, video inspection o		
0 4 was positively identified as broken	on July 1, 1983. Analysis of poter	ntial loss
o of hold-down force, loose parts, or	interference with CRA movement ind	icates that]
0 6 operation with broken hold-down spri	ngs does not affect the health and	safety of
0 7 the public.		
08		80
SYSTEM CAUSE CAUSE CODE CODE SUBCODE	COMPONENT CODE SUBCODE SUBCODE	
R C U E 12 B 13 13 13 13 13 13 15 SEQUENTIAL	F U E L X X (14) Z (15) Z (16)	REVISION
17 REPORT 8 3 0 1 3 21 22 23 24 25	ODE TYPE O 1 T T 31	NO.
Z 18 A 19 Z 20 Z 21 O 37	ATTA CHMENT NPRO-4 PRIME COMP. SUBMITTED FORM SUB. SUPPLIER Y 24 N 25	COMPONENT (26) MANUFACTURER 26 X 9 9 9 9
CAUSE DESCRIPTION AND COPRECTIVE ACTIONS (27) The cause of this occurrence was co		
failures were caused by fatigue ind		
1 2 which then propagated by fatigue.	The fourth spring (Mark B7) failur	e cause is
1 3 not yet known. The springs will be		
inspection program will continue, a	nd the Mark B7 spring failure will	be analyzed
TATUS STATUS SPOWER OTHER STATUS (30)	Video inspection of fuel	32 assemblies
7 8 9 10 12 13 44 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 1 6 Z 33 Z 34 NA	LOCATION OF RELEASE 36	
7 8 9 10 11 44 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39) 1 7 0 0 0 0 (37) Z (38) NA	45	
7 S 9 11 12 13 PERSONNEL INJURIES NUMBER DESCRIPTION (41)		80
1 8 0 0 0 40 NA	000700000 000704	80
LOSS OF OR DAMAGE TO FACILITY 43 TYPE DESCRIPTION NA	8307290033 830721 PDR ADDCK 05000269 S PDR	80
7 8 9 10 PUBLICITY ISSUED DESCRIPTION 45 2 0 N 44 NA	NR.	CUSE ONLY
NAME OF PREPARER Jocelyn C.	Petty PHONE: (704) 373	-8270

Duke Power Company Oconee Nuclear Station

Report Number: RO-269/83-13

Report Date: July 21, 1983

Occurrence Date: June 23, 1983

Facility: Oconee Unit 1, Seneca, South Carolina

Identification of Occurrence: Four fuel assembly broken hold-down springs discovered on Unit 1.

Conditions Prior to Occurrence: Refueling shutdown

Description of Occurrence: On June 23, 1983, three broken hold-down springs, and a possible fourth, were discovered during a reactor core scan just prior to refueling operations. This scan was performed to identify any loose parts or other unusual condition such as broken springs. The fourth spring required further viewing and was positively identified as broken on July 1, 1983.

Apparent Cause of Occurrence: The apparent cause of this occurrence was component failure. The first three springs in question were Mark B6 springs and were part of the same batch of assemblies that had experienced similar failures (as documented in RO-269/80-15, RO-270/82-03, and RO-287/82-07). The mechanism of failure for these three springs was fatigue induced cracking at an existing surface flaw which then propagated by fatigue. The fourth spring was a Mark B7 spring. The fatigue mechanism for this spring is still being evaluated. That analysis will be summarized in a later revision of this incident report as soon as the information becomes available.

Analysis of Occurrence: The safety implications of operation with broken hold-down springs were adequately addressed in the RO reports referenced above, and are repeated here. The spring failures pose three potential concerns: (1) loss of hold-down force; (2) loose parts; and (3) interference with normal CRA movement. Analysis of these potential concerns has eliminated any reasonable safety questions.

With regard to loss of hold-down force, Babcock and Wilcox has confirmed analytically that reactivity increases due to reinsertion of a "lifted" assembly adds less than 0.01% Ak/k per assembly. No lateral movement would result from lift since positive lateral restraint is provided through core internal structures. Lastly, no cyclic lifting/reinsertion is expected since one break in a spring reduces hold-down force slightly allowing the assembly to remain in place, and two breaks reduce the force to essentially zero, pinning the assembly in the lifted position.

Loose parts pose no additional safety hazard but are simply operational problems. All the spring breaks have occurred in one or both of the interface regions between the compressed and normal regions of the coil. Therefore, no more than three pieces, all of which exceed one complete circle, would be anticipated.

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Such large pieces would not escape the upper end fitting. If they did, normal core flow would sweep the parts to the OTSG upper head where the piece would be reduced to a size small enough to move through the tubes before reentering the core. Pieces of such size do not cause sufficient flow blockage to be a safety concern.

Preliminary analysis of worse case positioning of broken springs indicates that no configuration will allow sufficient force to prevent CRA insertion or to substantially increase drop time. There is no way for a piece to completely block the CRA path since the fingers are partially inserted in the guide tubes at all times.

The results of the safety analysis indicate that operation with broken hold-down springs does not affect the health and safety of the public.

Corrective Action: The Fuel Assembly Inspection Program will continue until future analysis of hold-down spring results indicates that such a program is no longer necessary. The broken fuel assembly hold-down springs will be replaced with Mark B10 springs prior to reloading them into the core. The Mark B7 broken spring will be examined by B&W to determine the failure mode. This information will be provided when available.

DUKE POWER GOMPANY P.O. BOX 33189 CHARLOTTE, N.C. 28242

HAL B. TUCKER VICE PRESIDENT NUCLEAR PRODUCTION

July 21, 1983

83 JUL 26 A8: 38

Mr. James P. O'Reilly, Regional Administrator U. S. Nuclear Regulatory Commission Region II 101 Marietta Street, NW, Suite 2900 Atlanta, Georgia 30303

Re: Oconee Nuclear Station Docket No. 50-269

Dear Mr. O'Reilly:

Please find attached Reportable Occurrence Report RO-269/83-13. This report is submitted pursuant to Oconee Nuclear Station Technical Specification 6.6.2.1.a(9), which concerns the discovery of conditions not specifically considered in the safety analysis report or Technical Specifications that require corrective measures to prevent the existence or development of an unsafe condition, and describes an incident which is considered to be of no significance with respect to its effect on the health and safety of the public. My letter of July 1, 1983 addressed the delay in preparation of this report.

Very truly yours,

Hal B. Tucker

JCP/php

Attachment

cc: Document Control Desk U. S. Nuclear Regulatory Commission Washington, D. C. 20555

> INPO Records Center Suite 1500 1100 Circle 75 Parkway Atlanta, Georgia 30339

> Mr. J. C. Bryant NRC Resident Inspector Oconee Nuclear Station

Mr. John F. Suermann Office of Nuclear Reactor Regulation U. S. Nuclear Regulatory Commission Washington, D. C. 20555

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