ATTACHMENT 1

Updates Included In This Submittal

Volume 3A

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EP OP-11, Revision 3

EP OP-38, Revision 4



CURRENT

EMERGENCY PLAN

IMPLEMENTING PROCEDURES

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REV

GSE Pa	cific Gas and Electric Company	NUMBER EP OP-11 REVISION 3
DEPARTMENT OF	NUCLEAR PLANT OPERATIONS	DATE 6/21/82
DIABLO CANYON	POWER PLANT UNIT NO(S) 1 AND 2	PAGE 1 OF 20
EMERGENCY	OPERATING PROCEDURE	IMPORTANT
TITLE LOSS OF C	OMPONENT COOLING WATER	TO
•		SAFETY
APPROVED:	R. C. PLANT MANAGER	DATE
	0	
CC005		
, B C D	CCW Inleakage - pg. 3 CCW Outleakage - pg. 5 Loss of CCW flow to the Letdown H	eat Exchanger -
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DIABLO CANYON POWER PLANT UNIT NO(S) 1 AND 2 .	NUMBER EP OP-11 REVISION 3 DATE 6/21/82 PAGE 2 OF 20
5. CCW VITAL HDR A/B (PK01-06)	
1) CCW Hdr Flo Lo 2) CCW Hdr Press Lo	
c. CCW Header C (PK01-08)	
1) CCW Hdr-C Flo Lo	
AUTOMATIC ACTIONS	
 Automatic start of STBY pump on low B. 	pressure in CCW headers A or
OBJECTIVES :	
1. Restore normal operation of CCW syst	tem.
ACTIONS/EXPECTED RESPONSE	RESPONSE NOT CETAINED
IMMEDIATE ACTIONS	
1. Verify STBY CCW Pump AUTO START 1.	Manual start STBY CCW Pump
SUBSEQUENT ACTIONS	
 If only one CCW pump can be 1. started, verify CCW heat exchanger outlet temperature is less than 95°F. 	Reduce system heat loads (as determined by the Shift Foreman) to maintain CCW system temperatures less than 95°F.
2. If <u>No</u> CCW pumps can be started,	
a. Trip the reactor.	
 b. Trip the reactor coolant pumps. 	
c. GO TO EP OP-5 Reactor Trip -with no Safety Injection.	

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	REVISION 4
DEPARTMENT OF NUCLEAR PLANT OPERATIONS	DATE 6/7/83
DIABLO CANYON POWER PLANT UNIT NO(S) 1 ANI	D 2 PAGE 1 OF 3
EMERGENCY PROCEDURE TITLE: ANTICIPATED TRANSIENT WITHOUT TRIP	(ATWT)
APPROVED RC Thoula	- 6/10/83
PLANT MANAGER	DATE

This procedure describes the steps to be taken in the event of an ATWT. An ATWT is a failure of the reactor protection system to trip the rods when a reactor trip setpoint has been exceeded. This procedure is "Important to Safety" and changes or revisions shall be approved by the PSRC.

SYMPTOMS

1.

2.

-3.

- 1. Reactor Trip setpoint exceeded without a reactor trip.
- Possible REACTOR PROTECTION SIGNAL Annunciator (PK 04-11 or PK 04-12) without a REACTOR TRIP ACTUATED Annunciator (PK 04-14)
- 3. DRPI indicates failure of rods to insert.
- -. NIS does not show a rapidly decreasing neutron count level.

ACTION/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
IMMEDIATE ACTIONS		
Manually Trip the Reactor. If the Reactor Trips GO TO EP OP-5.	1.	IF <u>NOT</u> , GO TO Step 2.
Manually Trip the Turbine	2.	Trip it locally at the turbine pedestal.
De-energize the M-G Sets by	3.	IF rods DO NOT insert,
13E.		a. Manually SI
		b. Insert Rods
		c. Locally open the Reactor Trip Breakers

d. GO TO EP OP-O

DIABLO CANYON POWER PLANT UNIT NO(S) 1 AND 2				NUMBER EP OP-38 REVISION 4 DATE 6/7/83 PAGE 2 OF 3	
TITLE	ANTI	CIPATED TRANSIENT WITHOUT TRIP	(ATWI	r)	
		ACTION/EXPECTED RESPONSE SUBSEQUENT ACTIONS		RESPONSE NO	T OBTAINED
	1.	Locally Open the Reactor Trip Breakers.	1.	Open Both M Generator M	MG Set Motor and/or Breakers.
	2.	Verify Feedwater isolation at T _{avg} < 554°F.	2.	Manually is feedwater a	solate main at T _{avg} < 554°F.
	3.	<u>Check AFW Status</u> a. AFW pumps running		a. Manuali	ly start AFW pumps.
		b. AFW Flow indicated or S/G NR Levels at 33%		b. Manual LCV's.	ly control AFW
	4.	When the Reactor Trip Breakers and/or Both MG Set Breakers are Open, Re-energize 13D and 13E.			
	5.	GO TO EP OP-5, Subsequent Step No. 1.			

DIABLO CANYON POWER PLANT UNIT NO(S)

1 AND 2

NUMBER EP OP-38 REVISION 4 DATE 6/7/83 PAGE 3 OF 3

TITLE: ANTICIPATED TRANSIENT WITHOUT TRIP (ATWT)

APPENDIX Z

EMERGENCY PROCEDURE NOTIFICATION INSTRUCTIONS

When this emergency procedure has been activated and upon direction from the Shift Foreman proceed as follows:

- Designate this event an <u>Alert</u>. Notify plant staff and response organizations required for this classification by Emergency Procedure G-2 "Establishment of On-Site Organization" and Emergency Procedure G-3 "Notification of Off-Site Organization" in accordance with Emergency Procedure G-1 "Accident Classification and Emergency Plan Activation."
- Designate this event a <u>Site Area Emergency</u> if safety injection was initiated before rods were inserted into the core but no core damage is evident (no abnormal increase in RCS coolant activity and no abnormal increase in gross failed fuel indication). Notify plant staff and response organizations required by EP G-2 and EP G-3 in accordance with EP G-1.
- Designate this event a <u>General Emergency</u> if <u>one</u> of the following conditions exist:
 - a. Core damage is evident by:
 - Reactor coolant activity greater than 300 µCi/cc equivalent I-131, or
 - Radiation levels indicate greater than 100% gap release (Refer to Appendix H of EP OP-1).
 - b. Complete loss of a safe shutdown system simultaneous with rods not inserted in the core.
 - c. Loss of CVCS capability to increase boric acid concentration in the RCS simultaneous with rods <u>not</u> inserted into the core.

Notify plant staff and response organizations required by EP G-2 and EP G-3 in accordance with EP G-1.

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IABLO CA	INVON POWER PLANT UNIT NO(S) 1 AND 2 NUMBER PLANT UNIT NO(S) 1 AND 2 NUMBER EP 0P-11 REVISION 3 DATE 6/21/82 PAGE 3 OF 20
. COM	PONENT COOLING WATER SYSTEM INLEAKAGE
1.	Surge tank level indicators reading high
2.	Possible Annunciator Alarms
	a. CCW SURGE TANK (PK01-07)
	1) CCW Surge TK Lv1 Hi
	b. CCW HEADER C (PK01-08)
	1) RCP Thermal Barrier CCW Flo Hi
	c. RCP (PK 05-01, 02, 03; 04)
	1) RCP Lower Brg Temp Hi 2) RCP No. 1 Seal Outlet Temp Hi
	d. HIGH RADIATION (PK 11-21)
	1) Process Monitor Hi-Rad (RE-17A&B)
AUT	OMATIC ACTIONS
1.	Surge tank vent closes on high radiation level.
2.	Reactor coolant pump thermal barrier return line isolation valve (FCV-357) closes on high flow.
OBJ	ECTIVES
1.	Prevent loss of primary water inventory into component cooling water system.
2.	Isolate component cooling water system to prevent radiation releases.
3.	Prevent damage to reactor coolant pumps.
	ACTIONS/EXPECTED RESPONSE RESPONSE NOT OBTAINED
	IMMEDIATE ACTIONS
1.	Verify CCW inleakage
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	ACTIONS/EXPECTED RESPONSE	RESPONSE NOT OBT	AINED
	IMMEDIATE ACTIONS CONTINUED		
	 Process monitor 17A&B reads one decade or more greater than monitor set point. 		
	 Possible increase in CCW Surge Tank level. 		
2.	Verify Surge Tank isolation valve RCV-16 closed if Hi Rad condition is verified.	2. CLOSE RCV-16 MANUA	ALLY (VB-1).
3.	If RCP thermal barrier return Tine isolates check adequate seal injection flow and normal Radial Bearing temperatures.	 If proper RCP Seal flows and Radial B temperatures canno maintained comment shutdown 	l Injection Bearing ot be ce a reactor
		a. If Hi Radial E Alarm temperat <u>THEN</u>	Bearing ture reached
		 Trip the React Trip the RCPs GO TO EP OP-5, Trip with no S Injection. 	tor , Reactor Safety
CA	JTION: Establish RCP seal cooling with tential introduction of steam into the RCP (Max rate of bearing coo down i	h CAUTION, in order to p CCW system and thermal 1°F/min).	prevent shock to
	SUBSEQUENT ACTIONS		
1.	Request a sample of the CCW system from the Chemical and Radiation Protection		

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	SUBSEQUENT ACTIONS CONTINUED
2.	Investigate to determine and isolate the source of leakage. Possible Sources:
	 a. Letdown Hx b. Thermal Barrier Hx c. Excess Letdown Hx d. RHR Hx e. Surge TK m/u valve failure (LCV-69 and/or LCV-70) f. NSSS Sample Coolers
3.	9. GPPD Sample Gobler Notify the Chemical and Radiation Protection Department prior to reopening RCV-16.
с.	COMPONENT COOLING WATER OUTLEAKAGE
c.	COMPONENT COOLING WATER OUTLEAKAGE
c.	COMPONENT COOLING WATER OUTLEAKAGE <u>SYMPTOMS</u> 1. CCW Surge Tank level indication decreasing
c.	COMPONENT COOLING WATER OUTLEAKAGE <u>SYMPTOMS</u> 1. CCW Surge Tank level indication decreasing 2. Possible Annunciator Alarms
c.	COMPONENT COOLING WATER OUTLEAKAGE <u>SYMPTOMS</u> 1. CCW Surge Tank level indication decreasing 2. Possible Annunciator Alarms a. CCW VITAL HDR A/B (PK01-06)
c.	COMPONENT COOLING WATER OUTLEAKAGE <u>SYMPTOMS</u> 1. CCW Surge Tank level indication decreasing 2. Possible Annunciator Alarms a. CCW VITAL HDR A/B (PK01-06) 1) CCW Hdr Press Lo
c.	COMPONENT COOLING WATER OUTLEAKAGE <u>SYMPTOMS</u> 1. CCW Surge Tank level indication decreasing 2. Possible Annunciator Alarms a. CCW VITAL HDR A/B (PK01-06) 1) CCW Hdr Press Lo b. CCW SURGE TANK (PK01-07)
с.	COMPONENT COOLING WATER OUTLEAKAGE <u>SYMPTOMS</u> 1. CCW Surge Tank level indication decreasing 2. Possible Annunciator Alarms a. CCW VITAL HDR A/B (PK01-06) 1) CCW Hdr Press Lo b. CCW SURGE TANK (PK01-07) 1) CCW Surge Tk Lv1 Lo
c.	COMPONENT COOLING WATER OUTLEAKAGE <u>SYMPTOMS</u> 1. CCW Surge Tank level indication decreasing 2. Possible Annunciator Alarms a. CCW VITAL HDR A/B (PK01-06) 1) CCW Hdr Press Lo b. CCW SURGE TANK (PK01-07) 1) CCW Surge Tk Lvl Lo 2) CCW Surge TK Make-up Vlv open
с. <u>А</u> Ш	COMPONENT COOLING WATER OUTLEAKAGE <u>SYMPTOMS</u> 1. CCW Surge Tank level indication decreasing 2. Possible Annunciator Alarms a. CCW VITAL HDR A/B (PKO1-06) 1) CCW Hdr Press Lo b. CCW SURGE TANK (PKO1-07) 1) CCW Surge Tk Lvl Lo 2) CCW Surge TK Make-up Vlv open <u>TOMATIC ACTIONS</u>

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DIABLO C	ANY	ON POWER PLANT UNIT NO(S) 1 AND 2		NUMBER EP 0P-11 REVISION 3 DATE 6/21/82 PAGE 6 OF 20
TITLE:	LO	SS OF COMPONENT COOLING WATER		
<u>c</u>	DBJE	CTIVES		
12	2.	Restore normal operation of component Prevent damage to vital equipment.	000	ling water system.
		ACTIONS/EXPECTED RESPONSE		RESPONSE NOT OBTAINED
		IMMEDIATE ACTION		
1	1.	Verify Surge Tank Makeup valves, LCV-69 and/or LCV-70 OPEN.	1.	OPEN CCW-62 and CCW-65 as necessary.
2	2.	Verify STBY CCW pump [®] AUTO STARTED on Loss CCW Hdr Pressure (Setpoint 46 psig Hdr A, 45 psig Hdr B).	2.	Manually START the STBY CCW pump if necessary.
3	3.	If Lined up to CCW System START Makeup Water Transfer Pump.	3.	Line up & START Makeup Water Transfer Pump to the CCW System.
		SUBSEQUENT ACTION		
1	1.	Verify makeup by observing increasing CCW Surge Tank level on LI-139 and LI-140.	1.	If there is <u>NO</u> Surge Tank level indication on LI-139 and LI-140.
				a. TRIP the Reactor
				b. TRIP the RCPs
				c. TRIP the CCW pumps
				d. GO TO EP OP-5 Reactor Trip with no Safety Injection.
2	2.	If the CCW pumps begin to cavitate (pump amps erratic), TRIP the CCW pumps.		
		a. TRIP the Reactor.		이 영화 영화 문서, 가지 않는
		b. TRIP the RCPs.		
		c. GO TO EP OP-5, Reactor Trip with no Safety Injection.		

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TLE:	L	OSS OF COMPONENT COOLING WATER		PAGE / OF 20
		ACTIONS/EXPECTED RESPONSE	RESPONSE	NUT OBTAINED
		SUBSEQUENT ACTION CONTINUED		
	3.	Determine the Source of the Outleakage and isolate. Refer to Appendix 1 of this procedure if a CCW header must be isolated due to a gross leak that is <u>otherwise</u> unsolable.		
	4.	If previously shutdown, restart the CCW pumps as soon as conditions permit.		
	5.	Shutdown the turbine building sump pumps as necessary.		
	6.	Refer to Emergency Procedure R-5 to combat any uncontrolled CCW coolant leakage.		
	7.	Notify the Chem & Rad Protection Department for assistance.		
	D.	LOSS OF CCW FLOW TO THE LETDOWN	HEAT EXCHANGER	
		SYMPTOMS		
		 Letdown temperature indicato normal. 	r reads higher th	an
		2. Possible Annunciator Alarms		
		a. LETDOWN PRESS/FLO TEMP (PK04-21)	
		1) Letdn Hx Outlet Temp	Hi	
		2) Letdn Outlet Temp Hi	Divert	

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DIABLO CAN	NYON POWER PLANT UNIT NO(S) 1 AND 2 NUMBER EP 0P-11 REVISION 3 DATE 6/21/82 PAGE 8 OF 20 OSS OF COMPONENT COOLING WATER
	AUTOMATIC ACTIONS
	 The letdown divert valve (TCV-149) bypass the letdown flow around the demineralizers to the volume control tank.
	OBJECTIVES
	 To <u>Isolate</u> Letdown path to Letdown Hx if CCW cannot be restored.
	 Maintain letdown and charging via alternate path until cooling is restored to letdown heat exchanger.
	ACTIONS/EXPECTED RESPONSE RESPONSE NOT OBTAINED
	IMMEDIATE ACTION
1.	Verify proper operation of TCV-130 1. Adjust TCV-130 manually to (CCW return from LTDN HX). establish adequate CCW flow.
	SUBSEQUENT ACTION
1.	IF CCW f.ow to LTDN HX CANNOT be restored THEN:
	a. Isolate LTDN - CLOSE CVCS 8149 A, B, and C.
	 Take manual control of charging flow.
	NOTE: Perform steps 1), 2), and 3) SIMULTANEOUSLY.
	1) Reduce charging flow rate.
	2) CLOSE HCV-142.
	 Maintain 32 gpm Seal Water injection to the RCPs (8 gpm per pump).
	c. CLOSE CVCS 8107 and 8108.
	d. Establish excess LETDN to the VCT.

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	SUBSEQUENT ACTION CONTINUED	1
2.	If the reciprocating charging pump is the pump in service, place its controller back in AUTO.	
E.	LOSS OF CCW FLOW TO REACTOR COOLING PUMPS <u>SYMPTOMS</u> 1. Thermal barrier and lube oil cooler cooling	water return high
	temperature indication. 2. Possible Annunciator Alarms	
	 a. CCW HEADER C (PK01-08) 1) RCP L.O. Clr CCW Flo Lo. 2) PCP Thermal Barrier CCW Flo Lo. 	
	b. RCP No (PK05-01, 02, 03, 04)	
	1) RCP Temp P250 2) RCP Lower Brg Temp Hi	
	AUTOMATIC ACTIONS	
	1. None	
	OBJECTIVES	
	 Prevent damage to reactor coolant pumps. 	
	ACTIONS/EXPECTED RESPONSE RESPONSE	ISE NOT OBTAINED
	IMMEDIATE ACTIONS	
* 1.	If CCW is lost to the lube oil coolers of <u>ONE</u> RCP.	
	a. Rapidly reduce power to 20% a. I THEN TRIP the RCP a	f RCP Hi Brg temperature larms sound:
	1) TRIP the RCP

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DIABLO CANYON POWER PLANT UNIT NO(S) 1 AND 2 NUMBER EP 0P-11 REVISION 3 DATE 6/21/82 PAGE 100F 20				
TITLE	OSS OF COMPONENT COOLING WATER			
	ACTIONS/EXPECTED RESPONSE	RESPONSE NOT OBTAINED		
	IMMEDIATE ACTIONS			
		 If the Reactor Trips GO TO EP-OP-5 Reactor Trip with no Safety Injection. 		
	 b. Take Manual control of affected loop S/G level, if necessary to stablize loop transient. c. Refer to Technical Specification for limitations on power operation with an idle loop. 			
2.	IF CCW is lost to MORE THAN ONE RCP:			
	a. TRIP the Reactor.			
	b. TRIP the affected RCPs.			
	c. GO TO EP OP-5, Reactor Trip with no Safety Injection.			
	SUBSEQUENT ACTIONS			
1.	If component cooling to the pump lube oil coolers is lost, leave the pump shutdown until cooling has been restored.			
2.	If cooling is lost to the thermal barrier:			
	 Check seal injection flow normal. 	a. Go to Section B, IMMEDIATE ACTIONS, step 3 (RESPONSE NOT OBTAINED)		
	b. Check radial bearing temperature normal.	 Go to Section B, IMMEDIATE ACTIONS, step 3 (RESPONSE NOT OBTAINED) 		
	c. Continue to run the pump(s) but increase surveillance.			
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TITLE	Ľ	OSS OF COMPONENT COOLING WATER			
		 If gross leakage from header C is suspected, go to Section C of this procedure. 			
	F.	LOSS OF CCW FLOW TO THE SEAL WATER	HEA	T EXC	CHANGER
		SYMPTOMS			
		1. Slight increase in Seal Inject	ion	tempe	erature.
		AUTOMATIC ACTIONS			
		1. None			
		OBJECTIVES			
		1. Restore normal Seal Water Heat	Exc	hange	er outlet temperature.
		ACTIONS/EXPECTED RESPONSE		RE	ESPUNSE NOT OBTAINED
		IMMEDIATE ACTIONS			
	1.	None Required			
		SUBSEQUENT ACTIONS			
	1.	Continue to operate. Reactor Coolant Pump temperatures will increase but not enough to damage equipment.	1.	If R or N exce limi resp follo	RCP radial bearing temperatur No. 1 Seal Leakoff temperatur eeds the normal operating ban its (alarms at 170°F and 160° bectively). Consider the lowing action:
				a.	Take manual control of TCV-130 & increase CCW flow to reduce VCT temperature.
				ь.	If excess LTDN is in service
,					1) Divert excess LTDN to th RCDT.
					 Verify AUTO makeup to th VCT.
				1.1	 Makeup manually if necessary.

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DIABLO CANYON POWER PLANT UNIT NO(S) 1 AND 2 .	NUMBER EP OP-11 REVISION 3 DATE 6/21/82 PAGE 12 OF 20
ACTIONS/EXPECTED RESPONSE SUBSEQUENT ACTIONS CONTINUED 2. Attempt to restore component cooling water flow.	RESPONSE NOT OBTAINED

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DIABLO CANYON POWER PLANT UNIT NO(S) I AND 2

NUMBER EP 0P-11 REVISION 3 DATE 6/21/82 PAGE 13 OF 20

TITLE: LOSS OF COMPONENT COOLING WATER

APPENDIX A

CLEARING A COMPONENT COOLING WATER HEADER DUE TO HEADER FAILURE

This procedure is intended to provide guidance for clearing a Component Cooling Water header due to a gross leak that is <u>otherwise</u> <u>unisolable</u>. The operators must determine which header the Teak is on and follow the steps below to clear the respective header.

- 1. Clearing Header "A"
 - a. If the leak is determined to be between the pump discharge and the component cooling water heat exchanger 1-1 motor operated outlet valve (FCV-430), check or place in service 1-2 component cooling water heat exchanger, then:
 - Close all three component cooling water pump discharge valves to header A.
 - Close component cooling water heat exchanger 1-1 motor operated outlet valve FCV-430.
 - Close the inlet valve to RE-17A (CCW 1-7).
 - Shut down the Turbine Building sump pumps as necessary.
 - Refer to Emergency Procedure R-5 and notify the Chemical and Radiation Protection Engineer.
 - b. If the leak is determined to be downstream of component cooling water heat exchanger outlet valve FCV-430, check or place 1-2 heat exchanger in service, then:
 - Close FCV-430.
 - Close CCW heat exchanger 1-1 outlet header A-C crosstie valve CCW 1-23.
 - Start 1-1 and 1-2 CCW pumps if available. Shutdown 1-3 CCW pump.
 - Close header A-C suction crosstie valve CCW 1-5.
 - 5) Isolate Makeup water to the "A" half of the surge tank.

NOTE: The following is a list of equipment normally served by header "A". Take appropriate actions to swap to alternate equipment.

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	APPENDIX A C CLEARING HEA Equipment normally serve	ADING "A" ad by CCW "A" header.	
	EQUIPMENT	ACTION	
	CCW pump 1-2 and 1-3 Stuffing Box and Lube Oil Coolers	Open "B" header supply and return for CCW pump 1-2 (CCW 1-104 and CCW 1-108). Close "A" header supply and return for 1-2 CCW pump (CCW 1-105 and CCW 1-109). NOTE: No action required for 1-3	
2.	Centrifugal Charging Pump 1-1 Lube Oil and Seal Coolers.	CCW pump since it was shut down. Start 1-2 or 1-3 charging pump if needed. Shut down 1-1 charging pump.	
3.	RHR Heat Exchanger 1-1 and RHR pump 1-1 Seal Water Cooler.	If in service, swap heat exchanger and pumps.	
¥.	Safety Injection Pump 1-2 Lube oil and Seal Water Coolers	Start 1-1 Safety Injection Pump if needed. Shut down 1-2 Safety Injection Pump.	
5.	Post LOCA Sample Coolers	None	

ITLE: L	055 01	F COMPONENT COOLING WATER
	c.	Shutdown the turbine building sump pumps as necessary. Refer to Emergency Procedure R-5 and notify the Chemical and Radiation Protection Engineer.
2.	Clea	aring Header "B"
	a.	If the leak is determined to be between the pump discharge and the component cooling water heat exchanger 1-2 motor operated outlet valve FCV-431, check or place in service 1-1 component cooling water heat exchanger, then:
		 Close <u>all</u> three component cooling water pump discharge valves to header "B".
		 Close component cooling water heat exchanger 1-2 motor operated outlet valve FCV-431.
		 Close the inlet valve to RE-17B (CCW 1-9).
		4) Shut down the Turbine Building sump pumps as necessary.
		5) Refer to Emergency Procedure R-5 and notify the Chemical and Radiation Protection Engineer.
	b.	If the leak is determined to be downstream of component cooling water heat exchanger outlet valve FCV-431, check or place 1-1 CCW heat exchanger in service, then:
		1) Close FCV-431.
		 Close CCW heat exchanger 1-2 outlet header B-C crosstie valve (CCW 1-24).
		 Start 1-2 and 1-3 CCW pumps if available. Shut down 1-1 CCW pump.
		 Close header B-C suction crosstie valve (CCW 1-4).
		5) Isolate makeup water to the "B" half of the surge tank.
		NOTE: The following is a list of equipment normally served by header "B". Take appropriate actions to swap to alternate equipment.

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P

DIABLO CANYON POWER PLANT UNIT NO(S) 1 AND 2

NUMBER EP 0P-11 REVISION3 DATE 6/21/82 PAGE 16 OF 20

TITLE: LOSS OF COMPONENT COOLING WATER

APPENDIX A CONTINUED

CLEARING HEADING "B"

Equipment normally served by CCW "B" header.

EQUIPMENT

ACTION

1. CCW Pump 1-1

Stuffing box and Lube Oil Coolers

 RHR heat exchanger 1-2 and RHR Pump 1-2 Seal Water Coolers

a.

Centrifugal Charging Pump 1-2
 Lube Oil and Seal Coolers

 Safety Injection Pump 1-1 Lube Oil and Seal Coolers

 Containment Fan Coolers 1-2 and 1-5. None required since 1-1 CCW pump was shut down.

If in service, swap RHR heat exchanger and pump to 1-1, and shutdown 1-2 RHR pump.

Start 1-1 or 1-3 Charging Pump if needed. Shut down 1-2 Charging pump.

Start 1-2 Safety Injection Pump if needed. Shut down 1-1 Safety Injection Pump.

Start 1-3 and 1-4 CFCU's. Shutdown-1-1, 1-2 and 1-5 CFCU's.

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DIABLO CAN	YON POWE	R PLANT UNIT NO(S) 1 ÁND 2	NUMBER EP 0P-11 REVISION 3 DATE 6/21/82 PAGE 17 OF 20
TITLE: LO	ISS OF CO	PONENT COOLING WATER	
	c. Sh	tdown the turbine building sump pu	umps as necessary.
	d. Re Ra	er to Emergency Procedure R-5 and liation Protection Engineer.	notify the Chemical and
3.	Clearin	Header "C"	
	NOTE: water t	learing Header "C" results in a lo all Reactor Coolant pumps.	oss of component cooling
	a. Pr	or to clearing header "C" reduce U	Init Load.
	b. Up	n clearing header "C":	
	1.	Trip the Reactor, then;	
	2.	Trip the Reactor Coolant pumps a Reactor Trip with no Safety Inje	and refer to EP OP-5
	3.	To clear header "C", perform the	following steps:
		a. Close header "C" supply FCV	-355.
		b. Start 1-1 and 1-3 CCW pumps	
	•	c. Shut down 1-2 CCW pump.	
		d. Close header A-C suction cr	osstie valve (CCW 1-5).
		e. Close header B-C suction cr	osstie valve (CCW 1-4).
		f. Close CCW-2, CCW pump 1-2 s	uction valve.
		g. Open DC control power local prevent an inadvertent AUTO closed.	ly to CCW pump 1-2 to) start with CCW-2
		NOTE: The following is a list o served by header "C" Unit 1. Ta to swap to alternate cooling wat necessary.	f equipment normally ke appropriate actions er or equipment as
			승규는 것이 같은 것을 많을 것이다.

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2	LUSS OF COMPONENT COULING WATE			
	APPENDIX A C CLEARING HEA	CONTINUED ADING "C"		
EQUIPMENT ACTION		ACTION		
	Reciprocating Charging Pump 1-3 Fluid Drive, Seal Plate, & Lube Oil Coolers	Start 1-2 or 1-3 Charging pump if needed. Shut down 1-3 Charging pump.		
	Letdown Heat Exchanger	Isolate normal letdown. Reduce charging flow. <u>Maintain Seal</u> Injection.		
	Seal Water Heat Exchanger	Monitor seal injection temperature		
	Excess Letdown Heat Exchanger	Unless it is in service, then isolate excess LTDN.		
	Spent Fuel Pit Heat Exchanger	Monitor spent fuel pool temperature		
	Steam Generator Blowdown Sample Coolers	NONE		
	Pressurizer Steam, Liquid and Hot Leg Sample Heat Exchangers	NONE		
	 a. Waste Concentrator b. Boric Acid Evaporator c. Auxiliary Steam Drain Receiver vent condenser 			

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ABLO	LOSS OF COMPONENT COOLING WATER	NUMBER EP 0P-11 REVISION3 DATE 6/21/82 PAGE 19 OF 20
	APPENDIX A CONT CLEARING HEADIN	INUED G "C"
	EQUIPMENT	ACTION
9.	 a. Reactor Vessel Support Coolers b. Gross Failed Fuel Detector Heat Exchanger c. Central Sample Panel Coolers 	NONE
10. Waste Gas Compressor 0-1 and 1-1 Seal Water Coolers.		Shut down 1-1 WGC and swap CCW supply and return valves to Unit Two CCW Header "C" for 0-1 WGC.

- 4. Shutdown the Turbine Building sump pumps as necessary.
- Refer to Emergency Procedure R-5 and notify the Chemical and Radiation Protection Engineer.

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 APFENDIX Z NOTIFICATION INSTRUCTIONS When this procedure has been activated and upon direction from the Shift Foremen, proceed as follows: Notify the Plant Superintendent, Supervisor of Operations and Plant Manager or their designate alternates, as a minimum within one how notify the NRC Bethessa Operations Center using the Red Phone tr the Control Room. Gather sufficient information from all course prior to calling so that the phone call is meaningful. Refer to Operating Procedure 0-4 "Operating Order (One Hour Reporting Requirements to NRC)" for a suggested format for reporting. If one vital loop of component cooling water is lost, designate this a <u>Notification of Unusual Event</u>. Notify plant staff and response organizations required for this classification of the Control Room. If one vital loops of component cooling water are lost, in mode mesons organizations in accordance with Emergency Plant Activation. If both vital loops of component cooling water are lost, in mode so for designate this an Alert. Notify plant staff and respons organizations required for the G-3 in accordance sints and event. If both vital loops of component cooling water are lost, in mode so for designate this an Alert. Notify plant staff and respons organizations required by ED G-2 and ED G-3 in accordance sints and event. If both vital loops of component cooling water are lost, in mode so for designate this an Alert. Notify plant staff and respons organizations required by ED G-2 and ED G-3 and ED G-3 and ED G-3 in accordance sints and event accordance with ED G-1. 	BLO CAN	SS OF	COMPONENT COOLING WATER	NUMBER EP 0P-11 REVISION 3 DATE 6/21/82 PAGE 20 OF 20
 NOTIFICATION INSTRUCTIONS 1. When this procedure has been activated and upon direction from the Shift Foremen, proceed as follows: a. Notify the Plant Superintendent, Supervisor of Operations and Plant Manager or their designated alternates, as a minimum vithin one hot notify the NRC Bethesda Operations Center using the Red Phone ir the Control Room. Gather sufficient information from all course prior to calling so that the phone call is meaningful. Refer to Operating Procedure 0-4 "Operating Order (One Hour Reporting Requirements to NRC)" for a suggested format for reporting. c. If one vital loop of component cooling water is lost, designate this a Notification of Unusual Event. Notify plant staff and response organizations required for this classification by implementing Emergency Procedures C-2." Establishment of the On-Site Emergency Organization" and G-3 "Notification of Off-Sit Organizations in accordance with Emergency Procedure G-1. d. If both vital loops of component cooling water are lost, in mode 5 or 6, designate this an Alert. Notify plant staff and respons organizations required by EP G-3 in accordance with EG G-1. e. If both vital loops of component cooling water are lost, in mode 1, 2, 3 or 4, designate this a Site Area Emergency. Notify plant staff and response organizations required by EP G-2 and EP G-3 and EP G-3 is accordance with EP G-1. 			ADDENDIX 7	
 Men this procedure has been activated and upon direction from the Shift Foremen, proceed as follows: Notify the Plant Superintendent, Supervisor of Operations and Plant Manager or their designated alternates, as a minimum. If any loss of component cooling water results in a reactor trip designate this a <u>Significant Event</u>. As a minimum within one how notify the NRC Bethesda Operations Center using the Red Phone ir the Control Room. Gather sufficient information from all course prior to calling so that the phone call is meaningful. Refer to Operating Procedure 0-4 "Operating Order (One Hour Reporting Requirements to NRC)" for a suggested format for reporting. If one vital loop of component cooling water is lost, designate this a Notification of Unusual Event. Notify plant staff and response organizations required for this classification of Uff-Sit Organizations" in accordance with Emergency Procedure G-1 "Accident Classification and Emergency Plan Activation." If both vital loops of component cooling water are lost, in mode 5 or 6, designate this a Alert. Notify plant staff and respons organizations required by EP G-2 and EP G-3 in accordance with E G-1. If both vital loops of component cooling water are lost, in mode 1, 2, 3 or 4, designate this a <u>Site Area</u> Emergency. Notify plant staff and response organizations required by EP G-2 and EP G-3 in accordance with EP G-1. 			NOTIFICATION INSTRUCTIONS	
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