

CONTROL BLOCK: [] [] [] [] [] [] (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 ALBRF 200-000000-000 4111 [] []

01 REPORT SOURCE L 05000260 012883 022583

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 During refuel outage, while loading fuel, the interlock that provides for rod withdrawal block became inoperable when the refuel platform was over the core. This resulted in a loss of rod withdrawal block function while the platform was over the core (TS 3.10.A.1). There are no redundant systems. Additionally, fuel loading procedures prevented any chance of inadvertent criticality. There was no effect on the health and safety of the public.

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP SUBCODE VALVE SUBCODE

17 LEW/RO REPORT NUMBER 83 11 E 12 A 13 INSTRU 14 S 15 Z 16 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Limit Switch #2 (GE Part #CR1156307) had failed because the plunger stuck due to wear holding the contacts closed and had resulted in the loss of the refueling interlock. Fuel loading was stopped and the refuel platform was moved from over the core. The limit switch was replaced and SI 4.10.A.1 was satisfactorily performed. No further corrective action is required.

15 FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)

16 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

17 PERSONNEL EXPOSURE NUMBER TYPE DESCRIPTION (39)

18 PERSONNEL INJURIES NUMBER DESCRIPTION (41)

19 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)

20 PUBLICITY ISSUED DESCRIPTION (45) 8303040433 830225 PDR ADOCK 05000260 S PDR

Tennessee Valley Authority
 Browns Ferry Nuclear Plant

Form BF 17
 BF 15.2
 2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50-260 / 83001 Technical Specification Involved 3.10.A.1

Reported Under Technical Specification 6.7.2.b.(2) * Date Due NRC 2/26/83

Event Narrative:

Unit 1 was operating at 88-percent power; unit 2 was in a refueling outage; and unit 3 was in cold shutdown. While loading fuel, the refuel interlock that provides for rod withdrawal blockage became inoperable with the refuel platform over the core. Fuel loading was stopped and the refuel platform was moved from over the core. Although no rods were pulled, the rod block function was inoperable while the refuel platform was over the core (Technical Specification 3.10.A.1). The interlock limit switch #2, GE Part #1156307 (which was replaced) had failed because of wear on the plunger, which resulted in the contacts being held in the closed position. After the replacement of the limit switch, Surveillance Instruction 4.10.A.1 was satisfactorily performed. However, Technical Specification Bases 3.10.A recognizes that the refueling interlocks are a backup to procedural core reactivity controls (fuel-loading procedure). There was no effect on the health and safety of the public.

No additional recurrence control or corrective action is required as this is considered a random failure.

* Previous Similar Events:

BFRO-50-259/7727
 50-260/8054
 50-296/8053

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: J.R.?