Appendix D

Scenario Outline

Facility:		Davis-Besse	Scenario No.:	3	Op Test No.:	DB NRC 2020	
Examiners:			Operato	ors: _ _ _		SRO ATC BOP	
Initial C	Conditio	ns: 100% Power					
Turnov		AS Ch 2 RCS Pres trument failure las	ssure LO and LOLO Bista t shift.	bles h	ave been trippe	d due to an	
Planne • •	 Planned: Stop #1 HDP for maintenance I&C to repair failed instrument and restore to service 						
Critical tasks: 1. Isolate overcooling SG (CT-17) 2. Restore Seal Return or Shutdown the Reactor Coolant Pumps (CT-B)							
Event No.	Malf. No.	Event Type*			vent cription		
1		N-BOP/SRO	Stop #1 Heater Drain	Pump	for maintenanc	e	
2		C-ATC/SRO	Makeup Filter High dif	f pres	s		
3		TS-SRO	Containment Spray Pu	ımp C)il Leak (TS)		
4		R-ATC/SRO C-BOP TS-SRO	RCP 1-1 high vibratior	ns — re	educe power – ti	rip RCP (TS)	
5		C-BOP/SRO	FW auto re-ratio fails	when	trip RCP		
6		C-ATC/SRO	Inadvertent SFAS Lev	els 1	through 3 – Trip	reactor	
7		Major	OVCLG - Leaking Mai	n Ste	am Safety Valve	•	
8		C-BOP/SRO	SFRCS fails to auto in	itiate	on Low Steam F	Pressure	
*	(N)ormal,	(R)eactivity, (I)nstrur	nent, (C)omponent, (M)ajor				

DAVIS-BESSE 2020 NRC SCENARIO 3

The crew will take the watch with power at 100%. SFAS Ch 2 RCS Pressure LO and LOLO Bistables have been tripped due to an instrument failure last shift. I&C will commence troubleshooting later this shift.

EVENT 1: Stop #1 HDP for maintenance

The crew will Stop #1 HDP IAW DB-OP-06227, Low Pressure Feedwater Heaters in preparation for planned PMs.

EVENT 2: Makeup Filter High diff press

After #1 Heater Drain Pump (HDP) has been stopped, the Lead Evaluator will cue event 2. Annunciator 2-4-A, Letdown or MU Filter Differential Pressure Hi, will alarm due to high differential pressure across the #1 Makeup Filter (PDI MU13 greater than 25 psid). The crew will implement alarm procedure DB-OP-02002, Letdown/Makeup Alarm Panel 2 Annunciators actions, validate #1 Makeup Filter differential pressure is high, and swap to #2 Makeup Filter using DB-OP-06006, Makeup and Purification System.

EVENT 3: Containment Spray Pump Oil Leak (TS)

The Lead Evaluator will then cue event 3. An EO performing normal rounds will call the control room and report the oil has leaked out of the Containment Spray Pump 1. The SRO will declare the Containment Spray Pump 1 Inoperable and enter TS 3.6.6 Condition A. The crew will direct disabling Containment Spray Pump 1 from starting.

EVENT 4 & 5: RCP 1-1 high vibrations- FW auto re-ratio fails when trip RCP

The Lead Evaluator will then cue RCP 1-1 high vibrations. Annunciators 6-1-A, MOTOR VIB HI and 6-5-A, MONITOR SYSTEM TRBL will alarm. The SRO will implement DB-OP-02515, RCP Pump and Motor Abnormal. The SRO will implement DB-OP-02504, Rapid Shutdown to reduce power. The ATC will reduce power to less than 72% and RCP 1-1 will be stopped. When the RCP is stopped FW will fail to auto re-ratio and the BOP will be required to take loop demands to hand and re-ratio. The CSRO will check RCS flow is greater than the flow required by TS 3.4.1, DNB Limits and Notify I&C to reduce the RPS High Flux Trip setpoints within 10 hours IAW TS 3.4.4, RCS Loops.

EVENT 6: Inadvertent SFAS Levels 1 through 3 – Trip reactor

After the plant has stabilized the Lead Evaluator will cue event 6. SFAS Channel 2 loses power causing SFAS levels 1 - 3 to actuate. An SFAS Level 3 inadvertent actuation is entry criteria for DB-OP-02000, RPS, SFAS, SFRCS Trip or SG Tube Rupture. SFAS Level 3 inadvertent actuation causes a loss of Seal Return to the RCPs. The 30-minute timer to restore Seal Return or SD the RCP's starts when RCP Seal Return is lost (CT-3).

EVENT 7 & 8: OVCLG - Leaking Main Steam Safety Valve & SFRCS fails to auto initiate

Following the reactor trip the crew will identify Overcooling due to a MSSV on SG 2 failing to reseat. The crew will implement attachment 20, lowering steam generator pressure attempting to seat the safety. The crew will recognize the Main Steam Safety Valve will not close and initiate and isolate SFRCS. SFRCS fails to automatically actuate on SG low pressure requiring manual re-positioning of the AFW valves. The crew will then isolate the OTSG and terminate the overcooling (CT-17). Due to the Inadvertent SFAS Actuation, Seal Return will be required to be restored within 30 minutes or the RCPs shutdown to prevent seal failure (CT-3).

When the overcooling steam generator is isolated, and Seal Return or RCPs shutdown is established the scenario can be terminated.

Appendix D	Operator Action Form ES-D-2
	cenario # <u>3</u> Event # <u>1</u> Page <u>1</u> of <u>1</u> top #1 HDP in preparation for planned PMs
Time Position	Applicant's Actions or Behavior
	ATER DRAIN TANK PUMPS 1 indicating lights DRAIN PUMP 1 (HDP 1) AMMETER
TEAM	Brief stopping Heater Drain Pump 1 IAW DB-OP-06227, Low Pressure Feedwater Heaters Section 3.5
TEAM	Verify adequate condensate pump capacity exists for the loss of the Heater Drain Pump flow.
ВОР	Direct EO to adjust HC311, FAIRCHILD RELAY LY311 HAND CONTROL for HDP 1 discharge valve, until the DIAPHRAGM PRESSURE TO LV311, is approximately 15 PSIG
BOOTH CUE	Role play as EO, DIAPHRAGM PRESSURE TO LV311, is approximately 15 PSIG
BOP	Stop Heater Drain Pump 1, using HIS318, LP HEATER DRAIN TANK PUMPS 1
BOP	Direct EO to Close HD 5, LOW PRESSURE HEATER DRAIN PUMP 1 DISCHARGE ISOLATION
BOOTH CUE	 Role play as EO, HD 5, LOW PRESSURE HEATER DRAIN PUMP 1 DISCHARGE ISOLATION is Closed Role play as I&C, remove from service and drain LSH391, LP FW HEATER DRAIN TK 1 IAW DB-OP- 06227 step 3.5.5.
ТЕАМ	Monitor Condensate Flow and Reactor Power.
TEAM	Evaluate expected Annunciators rection, cue Event 2, Makeup Filter High diff press

Appendix D	Operator Action Form ES-D-2
Op Test No.: 2020 S	Scenario # 3 Event # 2 Page 1 of 1
Event Description:	/lakeup Filter High diff press
Time Position	Applicant's Actions or Behavior
Indications Available:	
 PDI MU13 indic 2-4-A I FTDOW 	cates >25 PSID /N OR MU FILT ΔP HI
• 2-4-A LETDOW	
CREW	Recognize Makeup Filter high differential pressure for in service Makeup Filter 1 (indications available listed above)
	Implement DD OD 02002 Latdown/Makeun Alarm Danal 2
ATC	Implement DP-OP-02002 Letdown/Makeup Alarm Panel 2 Annunciators for 2-4-A LETDOWN OR MU FILT ΔP HI
ATC	Validate Makeup Filter high differential pressure with PDI MU13 indicating >25 psid
SRO	Direct placing Makeup Filter 2 in service in accordance with DB-OP-06006, Makeup and Purification System
SRO/ATC	Verify Makeup Filter 2 is not aligned as the Purification Demin Filter
SM CUE	<i>If necessary, Role-play Shift Manager and inform the SRO that Makeup Filter 2 has been verified <u>NOT</u> aligned as the <i>Purification Demin Filter</i></i>
ATC	Swap to Makeup Filter 2
ATC	Open MU12B, MAKEUP FILTER 2 INLET ISOLATION, using HISMU12B.
ATC	Close MU12A, MAKEUP FILTER 1 INLET ISOLATION, using HISMU12A.
CREW	Ensure work request initiated to have filter replaced (alarm procedure guidance)
On Lead Evaluator's	liscretion proceed to Event 3, MDFP Oil Leak

Appendix D			Operator Action					Form ES-D-2		
Op Test No.:	2020	Scenario #	3	Event #	3	Page	1	of	1	
Event Description:		Containme	nt Spra	ay Pump 1	Oil Leak	(TS)				
Time	Position			Applica	nťs Actions	or Behavior				

Indications Available:	
Local indicatio	n only, Equipment Operator report
Booth Cue	Role-play EO and report: "While performing my zone tour, I noticed that all the oil has drained out of Containment Spray Pump 1 inboard bearing reservoir. The sight glass is broken and will need replaced."
SRO	Declare Containment Spray Pump 1 Inoperable and enter TS
	3.6.6, Condition A
SRO	May contact Maintenance and/or Work Week Manager
Note	Crew may discuss/direct disabling Containment Spray Pump 1 from starting
Crew	May discuss/review plant risk with Containment Spray Pump 1 unavailable and discuss protecting equipment (Containment Spray Pump 2)
Booth Cue	If necessary, role play as maintenance and report "Containment Spray Pump 1 will be required to be removed from service to repair broken sight glass"
BOP	Turn on CS Pump Blue Operability Light
Booth Cue	 If necessary, role play as EO to disable CSP IAW DB-OP- 01000, OPERATION OF STATION BREAKERS: Remove the emergency close control power fuses behind the CSP 1 Emergency Control Transfer Switch on E1 upper right cubicle Place Emergency Control Transfer Switch for CSP 1 on E1 in EMERGENCY
On Lead Evaluator's c	liscretion, proceed to Event 4 & 5, RCP High Vibrations

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Appendix D		Operator Action	Form ES-D-2		
Op Test No.: Event Description:	<u>2020</u> So R	cenario # <u>3</u> Event # <u>4 & 5</u> Page CP High Vibrations and Auto FW Reratio Failure	<u>1</u> of <u>3</u>		
Time F	Position	Applicant's Actions or Behavior			
Indications Available: • 6-1-A MOTOR VIB HI • 6-5-A MONITOR SYSTEM TRBL					
	CREW	Recognize RCP 1-1 high vibrations			
	SRO	Implement DB-OP-02515, RCP and Motor Abno	rmal procedure		
	CREW	Verify the RCP 1-1 motor conditions exceed oper using the PPC or SPDS	erational limits		
	SRO	Implement DB-OP-02515 Attachment 1, Reactor Shutdown	r Coolant Pump		
	SRO	Implement DB-OP-02504, Rapid ShutdownCommence a power reduction to 72% or less			
	ATC	 <u>ULD with ICS in Automatic</u> At the LOAD CONTROL Panel, set the rate of rate specified by the Command SRO Select the target RTP as determined by the C 	-		
	CREW	Monitor Reactor Power to confirm power is being approximately the expected rate (NI vs. HBP)	g reduced at		
	ATC	Monitor Regulating Rod Insertion Limits during the per Technical Specification 3.2.1	he Shutdown		
	ATC	Attempt to maintain Axial Power Imbalance betw negative 20 percent using APSRs	een 0 and		
	BOP	Perform Attachment 6, Balance of Plant Actions Shutdown	for Rapid		

Α	bb	en	dix	D
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Op Test No.:	<u>2020</u> S	Scenario # <u>3</u> Event # <u>4 & 5</u> Page <u>3</u> of <u>3</u>
Event Descri	ption: F	RCP High Vibrations and Auto FW Reratio Failure
Time	Position	Applicant's Actions or Behavior
	BOP	At approximately 90 percent power, notify the Field Supervisor to remove the Auxiliary Feed Pump Turbine Main Steam Minimum flow lines from service
		When Condensate flow is less than 7.0 MPPH (FI578), then
	BOP	establish two Condensate pumps in operation
	SRO	Notify the System Control Center (SCC) Load Dispatcher of the unit load reduction
	BOP	Place the SG Load Ratio (ΔTc) in Auto
	ATC	Reduce reactor power to \leq 72 percent
	ATC	Stop RCP 1-1
	BOP	Verify proper Feedwater flow ratios of 2.4 to 1
		Recognize auto re-ratio is not occurring:
	BOP	Take both FW Loop Demands to Hand
		 Ratio FW to approximately 2.38 MPPH on SG1 Ratio FW to approximately 5.74 MPPH on SG2
<u> </u>		
	ATC	Verify Tave control is on RCS Loop 2
		Monitor Pressurizer level due to Tave transient
	ATC	4-2-E (Lo LVL) may alarm reduce MU-32 Setpoint to ~180"
		4-3-E (Hi LVL) may alarm refer to TS 3.4.9
	SRO	Check RCS flow is greater than the flow required by TS 3.4.1, DNB Limits. REFER TO DB-OP-03006, Miscellaneous
		Instrument Shift Checks. (Computer Point F744)

Appendix D	Form ES-D-2		
Op Test No.: 2020 So	cenario # <u>3</u> Event # <u>4 & 5</u> Page	3_ of _3	
Event Description: RCP High Vibrations and Auto FW Reratio Failure			
Time Position	Time Position Applicant's Actions or Behavior		
SRO	Notify I&C reduce RPS High Flux Trip setpoints	within 10 hours	
SRO	Enter TS 3.4.4 Condition A		
Proceed to Event 6 In:	advertent Actuation of SFAS Levels 1 through	3	

Appendix D		Operator Action Form ES-D-2		
Op Test No.: Event Descrip		cenario # <u>3</u> Event # <u>6</u> Page <u>1</u> of <u>1</u> advertent Actuation of SFAS Levels 1 through 3		
Time	Position	Applicant's Actions or Behavior		
• SF/ • SF/	s Available: AS annuncia AS compone M lights lit	tors nts repositioning		
*Critical Task (CT-B)	TEAM	Recognize an actuation of SFAS Levels 1 through 3 *Time of SFAS (Loss of RCP Seal Return)		
	ATC	 Perform DB-OP-02000 Immediate Actions Trip the reactor VERIFY Reactor Power is lowering on the Intermediate Range Trip the turbine VERIFY all Turbine Stop Valves OR Control Valves are closed 		
	US	Route to DB-OP-02000, Section 3		
	US/ATC	Verify Immediate Actions		
	TEAM	 Implement any necessary Specific Rules Actions ACTIONS FOR LOSS OF SUBCOOLING MARGIN STEAM GENERATOR CONTROL POWER FOR C1 AND D1 BUSES OR EDG START 		
	ATC	 Implement Specific Rule 6 Verify both EDGs are running Verify C1 and D1 are energized Verify both CCW Pumps are running Verify both Service Water Pumps are running 		
	ТЕАМ	 Implement any necessary Symptom Mitigation Sections LACK OF ADEQUATE SUBCOOLING MARGIN LACK OF HEAT TRANSFER OVERCOOLING 		
	Proceed to Events 7 & 8, Partially Stuck Open Main Steam Safety Valve and SFRCS failure to actuate on Low Steam Generator Pressure			

F					
Op Test No.:		cenario # <u>3</u> Event # <u>7 & 8</u> Page <u>1</u> of <u>3</u>			
Event Desch	Event Description: Partially Stuck Open Main Steam Safety Valve and SFRCS failure to actuate on Low Steam Generator Pressure				
Time	Position	Applicant's Actions or Behavior			
Indication	s Available:				
		n Generator Pressure			
	-	eam Pressure Actuation Fails			
	BOP	Recognize Overcooling due to SG 2 less than 960 psig			
	SRO	Route to DB-OP-02000, Section 7 for Overcooling			
	ATC	 Implement Attachment 8, Place MU/HPI/LPI in Service Start/Verify running both CCW Pumps Start both HPI Pumps Open HP 2A, HP 2B, HP 2C and HP 2D Start both LPI Pumps Open DH 64 and DH 63 Transfer Makeup Pump suctions, MU6405 & MU3971, to the BWST (depress off) Set Pressurizer Level Controller to 100 inches Start Standby Makeup Pump 2 IF AT ANY TIME Pressurizer Level is less than 40 inches Lock MU Pump suctions, MU6405 & MU3971, on the BWST 			
		 Verify Pressurizer heaters are off Isolate Letdown by closing MU2B Open MU6421 Throttle open MU6419 as needed (Alternate Injection Line) Direct EO to open MU6423B 			
	Booth	If security is called inform that steam continues to issue from any building roof steam relief exhaust pipe			
	Cue	from aux building roof steam relief exhaust pipe			
	BOP	 Perform Attachment 20 to attempt to reseat the MSSV Lower SG pressure toward 700 psig using the TBVs Recognize the MSSV has <u>NOT</u> reseated 			
	BOP	Initiate AND Isolate SFRCS using SFRCS MANUAL ACTUATION switches			
	BOP	Verify proper SFRCS alignment			

Appendix D)		Оре	erator Actic	on		Foi	m E	S-D-2
Op Test No.:	2020	Scenario #	3	Event #	7 & 8	Page	2	of	3
Event Descrip	•	•		iteam Safety enerator Pre		SFR	CS f	ailure	
Time			Applica	nt's Actions or	Behavior				

	CREW	Determine the overcooling cannot be isolated
	CREW	Determine SG 2 is causing the overcooling
Critical Task (CT-17)		NOTE: Overcooling must be isolated <10 minutes from the time SG 2 is less than 630 psig If applicable, Time SG 2 less than 630 psig
Critical		
Critical Task	BOP	Close AF 599 (isolate FW flow SG 2)
(CT-17)		Time AF 599 is closed
	BOP	Recognize AFW is not properly aligned for a low pressure trip on SG 2 andRealign AFW components as required
	BOP	Stabilize RCS temperature using the AVV on SG 1
	ATC	Initiate RCS depressurization to minimum adequate SCM
	ATC	Throttle MU/HPI as necessary to control Pressurizer level
	ATC	Restore Seal Injection using Attachment 10 Close MU 19 Block and open MU 66A, MU 66B, MU 66C and MU 66D Open MU 19 to obtain 12 - 15 gpm Seal Injection flow
	BOP	Recognize SFRCS SG level setpoint on High due to

Appendix D)		Оре	erator Actio	on		For	m E	S-D-2
Op Test No.:	2020	Scenario #	3	Event #	7 & 8	Page	3	of	3
Event Descrip	otion:		•		Steam Safety enerator Pre		SFR	CS f	ailure
Time	1		Applica	nt's Actions or	Behavior				

*Critical Task (CT-B)	ATC	Restore RCP Seal Return using Attachment 10 Verify MU 59A, MU 59B, MU 59C and MU 59D are closed *Block and open MU 38 *Block and open MU 59A, MU 59B, MU 59C and MU 59D OR *Stop RCPs Time MU38, MU59A, MU59B, MU59C and MU59E
		are opened or RCPs are stopped
	BOP	Verify proper SFRCS actuation
	CREW	Check for PTS criteria
	CREW	Check for adequate SCM

When Overcooling Steam Generator is isolated, and either Seal Return is restored or the Reactor Coolant Pumps are shutdown the scenario can be terminated

Justification for Critical Tasks

The term "Critical Task", is defined in NUREG-1021, OPERATOR LICENSING EXAMINER STANDARDS FOR POWER REACTORS. The listed critical tasks were compiled based on a review of Areva Technical Document 47-1229003-06, EOP Technical Bases Document, and additional CTs were added based on established DB Operations expectations and standards for previously identified Critical Task.

CT-17: Isolate Overcooling SG

- Safety Significance: The partially open Main Steam Safety Valve on Steam Generator 2 will cause an uncontrollable overcooling
- 2. Cues:
- Lowering Pressure on Steam Generator 2
- DB-OP-02000 Section 7 Overcooling
- 3. Measured by:
- Time zero for CT-17 is when Steam Generator 2 is less than 630 psig
- The required action to complete the critical task is to Close AF 599 (isolate FW flow SG 2)
- The required end time for completion of CT-17 is 10 minutes from when SG 2 is less than 630 psig
- 4. Feedback:
- Feedwater flow lowering
- AF599 indicates closed

CT-B: Protect RCP Seals

1. Safety Significance:

The inadvertent actuation of SFAS levels 1-3 will result in a loss of seal return flowpath. Extended operation without a RCP Seal Return flowpath may result in seal failure and unisolable RCS leakage.

- 2. Cues:
- Loss of seal return flow as indicated by MU38 closed and RCPs running
- DB-OP-02515, REACTOR COOLANT PUMP AND MOTOR ABNORMAL OPERATION and DB-OP-02000 Attachment 10
- 3. Measured by:
- Time zero for CT-B is when MU38 goes closed (inadvertent actuation of SFAS levels 1-3)
- The required action to complete CT-B is to restore Seal Return or shutdown RCPs
- The required end time for completion of CT-B is 30 minutes
- 4. Feedback: MU38, MU59A, MU59B, MU59C and MU59D are opened or RCPs are stopped

SIMULATOR SETUP INFORMATION

Simulator Setup:

- 100% Power
- Fail SFRCS low pressure trip on SG 2 automatic actuation
- Establish SG 2 unisolable steam leak following reactor trip

Procedures:

- DB-OP-06227, Low Pressure Feedwater Heaters
- DB-OP-02002, Letdown/Makeup Alarm Panel 2 Annunciators
- DB-OP-06006, Makeup and Purification System
- DB-OP-02006, RCP Alarm Panel 6 Annunciators
- DB OP 02515, RCP and Motor Abnormal
- DB-OP-02504, Rapid Shutdown
- DB-OP-02000, RPS, SFAS, SFRCS Trip or SG Tube Rupture
- DB-OP-01000, Operation of Station Breakers

For Simulator Instructor:

- DB-OP-06227, Low Pressure Feedwater Heaters
- Simulator Commands for disabling Containment Spray Pump 1 if required
 - BDP3H CSP1 LOCAL/REMOTE fuses TRUE
 - BDP3D CSP1 CLOSE FUSE 242 remove
 - BDP3E CSP1 CLOSE FUSE 243 remove

Page 1 of 3 SHIFT MGR/UNIT/FIELD SUPERVISOR/SHIFT ENGINEER TURNOVER CHECKLIST

DATE: Today	SHIFT: NIGHTS	<u>MODE: 1</u>	<u>POWER: 100%</u>
	SECURITY R	ISK: GREEN	
PLANT ONLINE RISK: GREE	<u>N</u>		PROTECTED TRAIN: 2
	PLANT CONI	DITION: 1	
ACTIVE ALERTS: NONE	<u>GRID CONDI</u>	TION: NORMAL	<u>SHUTDOWN DEFENSE-IN-DEPTH:</u> <u>N/A_</u>

ADDITIONAL PROTECTED EQUIPMENT – None CONTINGENCY PLANS FOR OPERABILITY/AVAIL IN EFFECT – None OUTSTANDING PROMPT OPERABILITY DETERMINATIONS – None

SHIFT ACTIVITIES -

Entered T.S. 3.3.5 Condition A.1 Safety Features Actuation System (SFAS) Instrumentation due to a failed RCS Pressure Transmitter I&C will commence troubleshooting on Day Shift Stop #1 HDP for maintenance

ALARM STATUS -None

ZONE 1 – ODMI TRIGGER POINTS -None ZONE 1 – WORKAROUNDS – None ZONE 1 – BURDENS – None

ZONE 2 – ODMI TRIGGER POINTS -None ZONE 2 – WORKAROUNDS – None ZONE 2 – BURDENS – None

ZONE 3 – ODMI TRIGGER POINTS -None ZONE 3 – WORKAROUNDS – None ZONE 3 – BURDENS – None

<u>CONTROL ROOM – ODMI TRIGGER POINTS</u>-None <u>CONTROL ROOM – WORKAROUNDS</u>-None <u>CONTROL ROOM – BURDENS</u> – None <u>CONTROL ROOM – DEFICIENCIES -</u>None

OUTAGE-RELATED repairs in the CTRM

GENERAL STATUS

Page 2 of 3 SHIFT MGR/UNIT/FIELD SUPERVISOR/SHIFT ENGINEER TURNOVER CHECKLIST

Intake Chlorination (Primary	Oxidant) – OOS	Intake Actibrom (Oxidant enhancer) - OOS			
Circwater Chlorination (Prim	ary Oxidant) – Pumps 1 & 3	Circwater Actibrom (Oxidant Enhancer) – I/S			
Circwater Trasar (Scale inhib	itor) – I/S Pumps 1 & 3	Collection Box Sodium Bisulfite (Dechlorinator) – I/S			
RCS Zinc Injection (Crud Layer Enhancement) – I/S		Polishers I/S: 1, 2, 3, 4	MUT press band: 30-40 psig		
MSD Skid #1: I/S	MSR 1 Drains: Forward	MSD Skid #2: I/S	MSR 2 Drains: Forward		

EVOLUTIONS IN PROGRESS – None

FOLLOW-UP ITEMS-None

Page 3 of 3 SHIFT MGR/UNIT/FIELD SUPERVISOR/SHIFT ENGINEER TURNOVER CHECKLIST

Date & Time Entered	Equipment Affected	Specification	Action Required	Next Action Due Responsible Individual	Issue Owner/ Test Leader
Today 0100	SFAS Ch 2 RCS Pressure Transmitter	T.S. 3.3.5 Condition A	A.1 One or more Parameters with one channel inoperable. Place channel in trip within 1 hour.	Action Completed	I&C

ACTIVE DB LICENSE REQUIREMENTS (TS, TRM, FHAR, ODCM, FLEX)

FUNCTIONAL EP RELATED EQUIPMENT REQUIRING COMPENSATORY ACTIONS

Appendix D

Facility:		Davis-Besse	Scenario No.: 4 Op Tes	t No.: DB NRC 2020		
Examin	ers:		Operators:	SRO ATC BOP		
Initial C	Initial Conditions: 100% Power					
		ain 100% Power				
Planne	d: Shift ro	outines				
	Shutdow	/n the Reactor - AT\ HPI (CT-5)	S (CT-24)			
Event No.	Malf. No.	Event Type*	Event Description			
1		N-BOP/SRO	Swap Cooling Tower Makeup Pump)S		
2		TS-SRO	RC TE3A6 Hot Leg temp to SCM m	eter (TS)		
3		C-ATC/SRO	MU32 fails closed (use alt inj line)			
4		C-BOP/SRO TS-SRO	Startup Transformer 02 Lockout (TS	3)		
5		C-BOP/SRO	MFP low oil pressure - trip MFP - ru	nback works		
6		R-ATC/SRO C-BOP	SG Tube Rupture			
7		Major	ATWS – Key Switch Works			
8		C-ATC/SRO	PZR Spray valve fails closed			
*	(N)ormal,	(R)eactivity, (I)nstrumen	(C)omponent, (M)ajor			

DAVIS-BESSE 2020 NRC SCENARIO 4

The crew will take the watch with the plant at 100% power.

EVENT 1: Swap Cooling Tower Makeup (CTMU) Pumps

The BOP will swap CTMU Pumps IAW DB-OP-06232, Circ Water and Cooling Tower Operation to support planned maintenance.

EVENT 2: RC TE3A6 Hot Leg temp to SCM meter fails high (TS)

The Lead Evaluator will cue event 2, RC TE3A6 Hot Leg temperature to SCM meter failure high. The crew will respond to annunciator 4-1-B, Sub Cooling Margin LO, and indications of RC TE3A6 failure, and SCM meter of 0 degrees. The SRO will identify this as a Tech Spec required Post Accident Monitoring instrument and enter TS 3.3.17 Condition A (TS).

EVENT 3: MU32 fails closed (use alt inj line)

When the Tech Spec 3.3.17 is entered the Lead Evaluator will cue MU32, Pressurizer Level Control Valve to fail closed. The crew will diagnose the failure with indications of low makeup flow and Pressurizer level lowering with MUT level rising. Manual control of MU32 demand signal will not affect makeup flow. The Unit Supervisor will implement abnormal procedure DB-OP-02512, Loss of RCS Makeup. MU32 will be isolated IAW DB-OP-06006. The alternate injection line may be placed in service at this time.

EVENT 4: Startup Transformer 02 Lockout (TS)

After letdown is in service, the Lead Evaluator will cue the Startup Transformer 02 Lockout. Annunciator 1-1-G, SU XFMR 02 LOCKOUT will alarm and the SRO will implement DB-OP-02001, Electrical Distribution Alarm Panel 1 Annunciators and enter T.S. 3.8.1 Off-Site AC Sources.

EVENT 5: MFP low oil pressure - trip MFP - runback works

When the T.S 3.8.1 is entered the Lead Evaluator will cue MFP low oil pressure. The BOP will implement DB-OP-02010, Feedwater Alarm Panel 10 Annunciators. The standby oil pump will fail to start which will require MFPT 2 to be tripped. The plant will runback to 55% power.

EVENT 6: SG Tube Rupture

At Lead Evaluators discretion a Steam Generator Tube Rupture will be cued. DB-OP-02000, Section 8 will be implemented. The Plant will be reduced to low level limits and steam flow will be transferred from the Turbine to the Turbine Bypass Valves.

EVENT 7: ATWS – Key Switch Works

After Steam Loads are transferred the Reactor Trip Buttons will fail resulting in an ATWS (CT-24). The ATC will activate the key switch which will shut down the reactor.

EVENT 8: PZR Spray valve fails closed

The ATC will be directed to reduce RCS pressure to minimum subcooling margin. When this is attempted it will be recognized that the PZR spray valve is failed closed and the alternate PZR Vent Line method will be used. The ATC will control HPI to maintain RCS inventory (CT-5)

When the pressure reduction has begun and HPI has been throttled the scenario can be terminated.

Appendix D)	Operator Action	Form ES-D-2
Op Test No.:	2020	Scenario # _4 _ Event # _1 Page	<u>1</u> of <u>1</u>
Event Descrip	tion:	Swap CTMU Pumps for maintenance	
Time	Position	Applicant's Actions or Behavior	
 HIS HIS HIS II922 	923, COOLIN 2 COOLING	G TOWER MAKEUP PUMP 1 indicating lights G TOWER MAKEUP PUMP 2 indicating lights FOWER MAKE-UP PUMP 1 AMMETER FOWER MAKE-UP PUMP 2 AMMETER	
	SRO	Direct BOP to Start #2 CTMU Pump and Stop #1 C	TMU Pump
	BOP	Start Cooling Tower Makeup Pump 2 using HIS923 TOWER MAKEUP PUMP 2	, COOLING
	BOP	Stop Cooling Tower Makeup Pump 1 using HIS922 TOWER MAKEUP PUMP 1	, COOLING
	TEAM	Monitor: • Motor amps • Cooling Tower Basin Level • Cooling Tower Make-up Flow	
	BOOTH CUE	<i>If required, role play as EO, pump indications ar pump discharge pressure is 34 psig and steady</i>	
At Lead Ev meter failu		ection, proceed to Event 2, RC TE3A6 Hot Leg ten	np to SCM

Appendix D	Operator Action	Form ES-D-2
Op Test No.: 20 Event Description:	D20 Scenario # 4 Event # 2 Page RC TE3A6 Hot Leg temp to SCM meter (TS)	<u>1</u> of <u>1</u>
Time Pos	ition Applicant's Actions or Behavior	
RC TE3A6 in	e: OOLING MARGIN LO dicates 899 °F nargin meter indicates 0 °F	
TEAM	Recognize indications of RC TE3A6 loop 2 Thot P	AM failure.
TEAM	 Refer to DB-OP-02004, Reactor Coolant Alarm Pa Annunciators, for 4-1-B Determines subcooling margin is satisfacto Determines RC TE3A6 loop 2 Thot instrum (TI RC3A6, Channel 2 Loop 2 Thot, fails high 	ent failure
SRO	Enter TS 3.3.17 Function 2, Condition A for RC TE Action - Restore within 30 days	3A6 Inoperable
SRO	May refer to TRM 8.3.7 for SCM meter – only 1 ch required	annel is
reference as a follow May refer to the follow DB-SC-03180 INSTRUMEN	NOTE: ure direction to reference TS 3.3.17. If necessary, as w-up. owing surveillance tests to evaluate operability of R D, REMOTE SHUTDOWN, POST ACCIDENT MONITO TATION MONTHLY CHANNEL CHECK 5, PSAT/TSAT MONTHLY TEST	RC TE3A6
On Lead Evaluator's	s discretion, proceed to Event 3, MU32 Fails closed	

Appendix D)	Operator Action Form ES-D-2
Op Test No.: Event Descrip		Scenario # 4 Event # 3 Page 1 of 2 MU32 Fails closed
Time	Position	Applicant's Actions or Behavior
	-	with no increase in makeup flow
	CREW	Recognize MU flow inappropriate for existing PZR level and MU32 demand signal
	CREW	Dispatch EO and/or I&C to MU32
	BOOTH CUE	After 2 minutes EO reports MU32 local position indicates closed, all other indications look normal. I&C does not see anything obvious, will get a work order started to troubleshoot
	SRO	Implement DB-OP-02512, Loss of RCS Makeup
	ATC	IF level is below set point by 20 inches then close MU2B using HISMU2B
	ATC	Reduce Letdown flow to a minimum of 25 gpm by throttling MU6 using HCMU6 and verify MU4 is closed
	ATC	Place MU32 in hand (LICRC14) and reduce demand to zero
		NOTE: use the guidance in DB-OP-02512 Attachment 4 to isolate ernate injection line in service.
	SRO	REFER TO DB-OP-06006, Makeup and Purification System, Section 4.20, Removal of MU32 from Service for Maintenance.
	ATC	Throttle MU6, LETDOWN FLOW CONTROL, as necessary to maintain a Pressurizer Level band of 210 to 220 inches.

Appendix D		Operator Action Form ES-D-
Op Test No.:	2020	Scenario # 4 Event # 3 Page 2 of 2
Event Descript		MU32 Fails closed
Time	Position	Applicant's Actions or Behavior
	ATC	Contact EO to Close MU209, NORMAL MAKE-UP FLOW CONTROLLER INLET ISOLATION.
	BOOTH	Role play as EO after 3 minutes, " MU209, NORMAL MAKE-
	CUE	UP FLOW CONTROLLER INLET ISOLATION is closed."
	ATC	IF additional MU flow is needed to control Pressurizer level THEN Raise the setpoint on FICMU19, RCP SEAL INJECTION FLOW CONTROL MU19, not to exceed a maximum of 10 GPM on any Reactor Coolant Pump
	SRO	Provide PZR level band for manual level control – May refer to DB-PF-06703 curve CC4.3
	SRO	REFER TO TRM 8.1.1, Boration Systems – Operating – No entry is required
	SRO	Notify SM to perform required notifications and Fleet Updates
	SRO	Direct restoration of Letdown flow to 70 gpm
	ATC	Throttle MU6 using controller HCMU6 to restore Letdown flow to 70 gpm
	wn is in ser nsformer Lo	vice and at Lead Evaluator's direction, proceed to Event 4, occourt

Appendix D Operator Action Form							
Op Test No.: Event Descrip	_2020 tion:	Scenario # 4 Event # 4 Page 1 of 2 Startup Transformer Lockout					
Time Position Applicant's Actions or Behavior							
 1-1-G S 1-2-G S ACB 34 	S Available: U XFMR 02 L U XFMR 02 I 562 Open 5 Open & 81-I	DNGR					
	BOP	Implement DB-OP-02001, Electrical Distribution Panel 1					
	CREW	Recognize Startup Transformer 02 Lockout and loss of K bus					
	BOP	Check Bus A and B are energized					
	ATC/BOP	Verify the following are open: • 34562 • 81-B-65 • 81-B-67 • HX02A • HX02B					
	SRO	Notify the Load Dispatcher K Bus is de-energized					
	Booth Cue	Role play as Load Dispatcher: "We are sending a crew to investigate"					
	SRO	Direct Isolation of SU Transformer 02 Refer to DB-OP-06311, 345KV Switchyard Procedure 					
	Booth Cue	Role Play as the Field Supervisor, "Isolate SU Transformer 02 IAW DB-OP-06311"					
	BOP/ATC	Dispatch Equipment Operator to investigate S/U Xfmr 02					

Appendix D		Operator Action Form ES-D-2				
Op Test No.:		Scenario # _4 Event # _4 Page _2 of _2				
Event Descript	tion:	Startup Transformer Lockout				
Time	Position	Applicant's Actions or Behavior				
	Booth Cue	Role play as Equipment Operator: "Local Annunciator Alarm (102-7-A) SUDDEN PRESSURE is in alarm. SU Transformer 02 has no visible damage."				
	SRO	Implement DB-OP-02521, Loss of AC Bus Power Sources				
	SRO	Notify the Shift Manager to perform the following: • REFER TO RA-EP-01500, Emergency Classification. • REFER TO NOP-OP1003, Grid Reliability Protocol. • REFER TO NOP-OP-1015, Event Notifications. • REFER TO NOBP-OP-0011, Fleet Reporting and Updates.				
	SRO	 Refer to Tech Spec 3.8.1, AC Sources – Operating Enter TS 3.8.1, Condition A Direct performance of DB-SC-03023, Off-Site Sources 				
	SM Cue	Another Operator will ensure completion and review of DB- SC-03023, Off-Site Sources				
	SRO	Inform the System Dispatcher to take all necessary steps to restore power to 02 Startup Transformer				
	ATC/BOP	Refer to DB-OP-02102, Startup Transformer 02 Alarm Panel 102 Annunciators for alarm 102-7-A (same actions as transformer lockout)				
	BOP	Transfer B bus reserve source selector switch to X01				
	Booth Cue	<i>If necessary, role play as the Field Supervisor, "IAW DB-OP- 06311, Step 3.17.A, verify HS6295, BUS B RESERVE SOURCE, in the X01 position."</i>				
When Tech	Spec is enf	ered and On Lead Evaluator's direction, proceed to Event 5,				
		trip MFP, runback works				

Appendix D		Operator Action Form ES-D-2
Op Test No.: Event Descriptio		Scenario # <u>4</u> Event # <u>5</u> Page <u>1</u> of <u>1</u> MFPT low oil pressure - trip MFP, runback works
Time	Position	Applicant's Actions or Behavior
		UBE OIL PRESS LO ESS
	BOP	Implement DB-OP-02010, Feedwater Alarm Panel 10 Annunciators
	BOP	Determine Bearing Pressure is low using PI1256, HDR PRESS
	BOP	Attempt to start standby MFPT 2 Main Oil Pump
	BOP	Start MFPT 2 Emergency Bearing Oil Pump using HIS1188, EMER BEARING OIL PUMP
	BOP	Trip MFPT 2 using HS798, TURBINE TRIP
	CREW	Monitor Plant Runback to 55% power
	SRO	REFER TO DB-OP-06401, Integrated Control System Operating Procedure for plant runback
	SRO	REFER TO DB-OP-06902, Power Operations, for guidance to operate plant equipment for the current power level
	BOP	Dispatch Zone 1 Operator
	Booth Cue	No Indication of oil leaks at MFPT 2
	SRO	Enter TS 3.4.9 Condition A IF Pressurizer water level > 228 inches
•		bilized and at Lead Evaluator's direction, proceed to Event 6, Rupture with ATWS

Appendix D		Operator Action Form ES-D-2					
Op Test No.:	2020	Scenario # _4 _ Event # _6, 7 & 8 _ Page _1 _ of _4					
Event Descrip	Event Description: Steam Generator Tube Rupture with ATWS and Spray valve failed closed						
Time	Position	Applicant's Actions or Behavior					
	A !! . .						
Indications	MN STM LIN						
	AC SYS DIS						
-		idenser off-gas radiation monitors					
-	tivity on eith ing Makeup	er main steam line radiation monitor flow					
	CREW	Recognize indications of a tube leak #2 Steam Generator					
		5					
	SRO	Implement DB-OP-02531, Steam Generator Tube Leak					
	300	Implement DD-OF-02331, Steam Generator Tube Leak					
		Only date the DOO has been to using the second state of the form					
	ATC	Calculate the RCS Leak rate using the computer calculation for RCS Leakage. (5 minute average preferred)					
	SRO	Route to DB-OP-02000, SFAS, RPS, SFRCS Trip or SG Tube Rupture, Section 8					
	ATC	Monitor Pressurizer level					
	AIC	Anytime Pressurizer level is <100 inches – trip Reactor					
	SRO	Notify the Shift Manager to refer to RA-EP-01500, Emergency Classification					
	BOP	 Implement Attachment 8, Place MU/HPI/LPI in Service Start the STBY CCW Pump Start both HPI Pumps Open HP 2A, HP 2B, HP 2C and HP 2D Start both LPI Pumps Open DH 64 and DH 63 Transfer MU Pump suctions to the BWST AND press OFF Start the STBY Makeup pump 					
		May place Alternate Injection Line in-service					
	ATC	If Pressurizer <200 inches isolate Letdown – Close MU2B					

Appendix D		Operator Action Form ES-D-2			
Op Test No.: 2020 Event Description:		Scenario # 4 Event # 6, 7 & 8 Page 2 of 4 Steam Generator Tube Rupture with ATWS and Spray valve			
		failed closed			
Time	Position	Applicant's Actions or Behavior			
	I	l			
	ATC	Place SG/RX in Hand and reduce power to place both SGs on low level limits			
	SRO	Direct Chemistry to perform Attachment 2			
	SRO	Direct Radiation Protection to perform Attachment 3			
	SRO	Direct an Equipment Operator/Field Supervisor to perform Attachment 4			
	SRO	Direct startup of the Auxiliary Boiler on Demineralized water and then transfer Auxiliary Steam to the Auxiliary Boiler			
	Booth	<i>If necessary, role play as EO, " Aux Boiler start-up in progress."</i>			
	Cue	NOTE: Booth operator to prevent Aux Boiler Trip			
		When both SGs on low level limits			
	ATC/BOP	 Verify SG/RX Demand at zero demand (ATC) Place BOTH SG Feedwater Demands ICS stations in HAND AND reduce the demands to zero Place Turbine Bypass valves in Hand Reduce Generator load to less than 50 MWe 			
	ATC	Attempt to Trip Reactor at 50 MWe			
Critical Task (CT-24)	ATC	 Recognize the reactor does not trip and perform DB-OP-02000, Immediate Actions response not obtained At the Rod Control Panel, insert the Reactor Trip Test key <u>AND</u> rotate clockwise to depower the CRDMs 			

			~	(A ()			_		0 0 0
Appendix D			Ope	rator Actio	n		For	m E	S-D-2
Op Test No.:	_2020_S	cenario #	4	Event #	6,7&8	Page	3	of	4
Event Description:		team Generalied closed		Tube Rup	ture with ATV	/S and Sp	ray v	alve	
Time	Position Applicant's Actions or Behavior								

ATC	Complete DB-OP-02000 Immediate Actions Verify power decreasing in the intermediate range Manually trip the turbine
CREW	Implement Specific Rules
CREW	Check for Symptoms
SRO	Route to Section 8 for SG Tube Rupture
BOP	Verify Attachment 8 is complete
BOI	
ATC	If Pressurizer Level is less than 40 inches: • Lock MU Pump suctions on the BWST • Isolate Letdown • Verify Pressurizer heaters are off
ATC	Verify Pressurizer level controller set at 100 inches NOTE: MU 32 failed closed, may use MU 32 Bypass and/or place Alternate Injection Line in service
SRO	 Verify DB-OP-02531 Attachments are in progress Direct Chemistry to perform Attachment 2 Direct Radiation Protection to perform Attachment 3 Direct an Equipment Operator/Field Supervisor to perform Attachment 4
ATC	Turn off all Pressurizer heaters
1	
	CREW CREW SRO BOP ATC ATC

Appendix D		Operator Action Form ES-D-
Op Test No.: Event Descriptio		Scenario # <u>4</u> Event # <u>6, 7 & 8</u> Page <u>4</u> of <u>4</u> Steam Generator Tube Rupture with ATWS and Spray valve failed closed
Time	Position	Applicant's Actions or Behavior
	SRO	Route to step 8.25 for using the Pressurizer Vent for RCS pressure reduction
	ATC	Start the Quench Tank Circ Pump
	ATC	Close DR2012A and DR2012B
	ATC	Block SFAS low RCS pressure trips when Block Permits light
	ATC	 Reduce RCS pressure to close to the minimum adequate SCM: Pressurizer Vent Line Method Open RC 200 Open RC 239A Cycle RC 239A and Pressurizer Heaters to control RCS Pressure
Critical Task (CT-5)	ATC	Throttle High Pressure InjectionControl Pressurizer level by controlling MU and HPI
	CREW	Check for PTS Criteria, REFER TO Specific Rule 5
	BOP	Begin an RCS cooldown AND depressurization at 100°F/hr, using both SGs
		eduction and Cooldown have begun, and HPI has been an be terminated

Justification for Critical Tasks

The term "Critical Task", is defined in NUREG-1021, OPERATOR LICENSING EXAMINER STANDARDS FOR POWER REACTORS. The listed critical tasks were compiled based on a review of Areva Technical Document 47-1229003-06, EOP Technical Bases Document, and additional CTs were added based on established DB Operations expectations and standards for previously identified Critical Task.

CT-24: Shutdown Reactor- ATWS

- Safety Significance: When the Reactor is tripped the reactor trip pushbuttons will fail to cause a reactor trip.
- 2. Cues:
 - Reactor power not lowering on the Intermediate Range
 - DB-OP-02000, Perform RNO IAs for ATWS (Reactor Trip Test Key turned clockwise)
- 3. Measured by: Continuing in EOP with mitigation strategy prior to reactor being tripped
- 4. Feedback: Reactor Power is lowering on the Intermediate Range

CT-5: Control High Pressure Injection

1. Safety Significance:

A Steam Generator Tube Rupture will require High Pressure Injection to be placed in service and throttled to prevent violating the RPV P-T Limit by maintaining RCS inventory and minimum subcooling margin

- 2. Cues:
 - SCM Meters
 - HPI Flow
 - DB-OP-02000 Specific Rule 3
- Measured by: HPI must be throttled prior to violating RV P-T Limit
- 4. Feedback:
 - SCM Meters
 - Pressurizer level
 - RCS MU Flow

• SIMULATOR SETUP INFORMATION

- 1. Simulator Setup
 - 100% Power
 - ATWS (Key switch works)
 - PZR Spray valve failed closed on Rx trip
 - #1 CTMU in service

2. Procedures

- DB-OP-06232 Circ Water and Cooling Tower Operation
- DB-OP-02004, Reactor Coolant Alarm Panel 4 Annunciators
- DB-OP-02512 Makeup and Purification System Malfunctions
- DB-OP-02001, Electrical Distribution Alarm Panel 1 Annunciators
- DB-OP-02521, Loss of AC Bus Power Sources
- DB-OP-06311, 345KV Switchyard Procedure
- DB-OP-02102, Startup Transformer 02 Alarm Panel 102 Annunciators
- DB-OP-02010, Feedwater Alarm Panel 10
- DB-OP-06401, Integrated Control System
- DB-OP-06902, Power Operations
- DB-OP-02531, Steam Generator Tube Leak
- DB-OP-02000, RPS, SFAS, SFRCS Trip or SG Tube Rupture
- 3. For Simulator Instructor:
 - None

SHIFT MGR / UNIT/FIELD SUPERVISOR/ SHIFT ENGINEER TURNOVER CHECKLIST

DATE: Today	SHIFT: NIGHTS	MODE: <u>1</u>	POWER: <u>100</u> %
	PROTECTI	ED TRAIN: <u>2</u>	
PLANT RISK: <u>GREEN</u>	<u>N</u> SECURITY RI	SK: <u>YELLOW</u>	GRID RISK: <u>GREEN</u>
LICENSE REQUIREMENT	<u>-S:</u>		

CONTINGENCY PLANS IN EFFECT:

OUTSTANDING OPERABILITY EVALUATIONS:

ALARM STATUS:

AT THE CONTROLS RO / ZONE 3 EO

CONTROL ROOM LEVEL 1 WORK AROUNDS

CONTROL ROOM LEVEL 2 WORK AROUNDS

CTRM DEFICIENCIES

<u>GENERAL STATUS</u> Maintain 100% power Shift Routines Swap CTMU in preparation for planned maintenance

ZONE 3 – LEVEL 1 WORKAROUNDS ZONE 3 – LEVEL 2 WORKAROUNDS

GENERAL NOTES

BALANCE OF PLANT RO / ZONE 1 EO / ZONE 2 EO

GENERAL STATUS

ZONE 1 – LEVEL 1 WORKAROUNDS ZONE 1 – LEVEL 2 WORKAROUNDS

ZONE 2 – LEVEL 1 WORKAROUNDS ZONE 2 – LEVEL 2 WORKAROUNDS

<u>GENERAL NOTES</u> CCW 1 – 85 °F CCW 2 – 85 °F CCW 3 – 110 °F

Command SRO / Field Supervisor / Shift Engineer Input

CHEMISTRY – STATUSIntake Chlorination – Bays 2 & 3Circwater Chlorination – 1 & 3Circwater Dynacool – I/SSodium Bisulfite – I/SMSD Skid \Rightarrow I/SMSR Drains: #1 \Rightarrow Forward#2 \Rightarrow ForwardPolishers I/S – 1,2,3,4

Evolutions in Progress

Follow-up Items

M&TE Issued for Testing- NOP-WM-5002 Compliance:

EPE 104

Page 2 SHIFT MGR / UNIT/FIELD SUPERVISOR/ SHIFT ENGINEER TURNOVER CHECKLIST

Specification	Equipment Affected	Date & Time Entered	Action Required	Issue Owner/ Test Leader	Next Action Due/ Responsible Individual

Appendix D

Scenario Outline

Facility:		Davis-Besse	Scenario No.: 5	Op Test No.:	DB NRC 2020	
Examin	ers:		Operators:		SRO	
					ATC	
					BOP	
		4000/ 5				
Initial C	Conditio	ns: 100% Power				
Turnover: Maintain 100% Power						
Planne	d: Rout	ine activities				
Critical	tasks:					
1.	Isolate	Overcooling SG (CT-1	7)			
		ow Control (CT-16) I RCS Inventory (CT-30))			
	-	-) (- ,			
Event	Malf.	Event Type*		vent		
No.	No.		Des	cription		
1		R-ATC/SRO TS-SRO	D2 Lockout - Power redu	iction (TS)		
2		C-BOP/SRO	SAC 2 trip (SAC 1 and E	IAC Auto start fai	l)	
3		I-ATC/SRO	Power range (NI5) high f	ailure		
4		I-BOP/SRO TS-SRO	Place RPS Channel 2 in	Manual Bypass (TS)	
5		C-ATC/SRO	Turbine Vibration - trip R	x		
6		Major/ALL	OVCLG - MSR 2ND STO	6 FAIL		
7		C-BOP/SRO	AFP Target rock fails ope	en - SG overfeed		
8		C-ATC/SRO	MUP Trip – Loss of Both	MUPs		
*	(N)ormal,	(R)eactivity, (I)nstrument,	(C)omponent, (M)ajor			

DAVIS-BESSE 2020 NRC SCENARIO 5

Direction for the crew is to maintain 100% power.

EVENT 1: D2 Lockout - Power reduction due to loss of Cond Pump (TS)

The lead evaluator will cue event 1, D2 Lockout. The CSRO will implement DB-OP-02521, Loss of AC Bus Sources. The crew will identify a loss of #2 Cond Pump. DB-OP-02504, Rapid Shutdown will be utilized to lower power to within the capacity of two condensate pumps.

EVENT 2: SAC 2 trip (SAC 1 and EIAC Auto start fail) (TS)

SAC 2 trip will also lose power when Bus D2 is lost. SAC 1 and EIAC fail to Auto start causing entry into DB-OP-02528, Loss of Instrument Air. The BOP will manually start SAC 1 and verify IA pressure recovers. T.S. 3.7.5 for the Motor Driven Feed Pump will be entered.

EVENT 3: Power range (NI5) high failure

When T.S. 3.7.5 has been entered, the lead evaluator will cue event 3, Power Range Nuclear Instrument NI-5 fails high, which will cause rods to insert due to the ICS system responding to the failed high signal. The crew will take actions to stabilize the plant per abnormal procedure DB-OP-02505, Nuclear Instrument Failures.

EVENT 4: Place RPS Channel 2 in Manual Bypass (TS)

When the plant is stable, the Unit Supervisor will direct placing RPS Channel 2 in Manual Bypass IAW DB-OP-06403, Reactor Protective System (RPS) and Nuclear Instrumentation (NI) Operating Procedure, and the Power Range Test Module in Test Operate IAW DB-OP-02505, Nuclear Instrumentation Failures. The Unit Supervisor will enter applicable T.S. 3.3.1 for RPS Channel 2 in Manual Bypass.

EVENT 5: Turbine Vibration - trip Rx

The Lead Evaluator will cue the Main Turbine bearing vibration increase. Annunciator 15-2-E, T-G BEARING VIB HI will alarm. The crew will respond in accordance with DB-OP-02015, Turbine Alarm Panel Annunciators, trip the reactor and GO TO DB-OP-02000, RPS, SFAS SFRCS Trip or SG Tube Rupture.

EVENT 6: OVCLG - MSR 2ND STG FAIL

Event 6 will be automatically triggered when the reactor trips. The MSR Second Stage Reheat Valves fail to auto close. The crew will identify indications of Overcooling and the Unit Supervisor will route to section 7 for Overcooling. Implementation of attachment 20, Isolate or Control Potential Source of Overcooling, should identify the MSR Second Stage Reheat Valves failing to auto close. MS314 and 199 will not close if attempted to manually close. The crew will then Initiate and isolate SFRCS to terminate the overcooling. (CT-17)

EVENT 7: AFP Target rock fails open - SG overfeed

AFPT 2 will start and feed at full flow with its discharge valve failed open. The BOP will be required to control SG2 level with AFPT 2 speed control. (CT-16)

EVENT 8: MUP Trip – Loss of Both MUPs

The Makeup pump (MUP) will trip due to low lube oil pressure at the same time as the Reactor Trip. The STBY MUP will not start. RCS Pressure will be lowered to allow HPI flow to maintain RCS Inventory. (CT-30)

When SG2 level and the Pressurizer level are being controlled the scenario can be terminated.

Appendix D		Operator Action			Form ES-D-2		
Op Test No.:	2020 Scenario #	5 Event#	1 and 2	Page	1 of	2	
Event Description:		wer Reduction SAC			<u> </u>		
Time Positi	on	Applicant	's Actions or Beha	ivior			
Indications Availa • 0 volts on • All D2 Bus • AD213 Opt	El6261 load and supply	breakers open					
Tea	n Recognize [D2 bus has locke	ed out				
SRO	D Route to DB	8-OP-02521, Los	s of AC Bus So	ources			
SRO	• REFER TO • REFER TO	hift Manager to p) RA-EP-01500,) NOP-OP-1015) NOBP-OP-001	Emergency Cl , Event Notifica	assificatio ations.			
TEA	M Review E-10 loads have b	040A, Electrical been lost	Distribution Ma	nual to de	termine	what	
TEA	M Recognize le	oss of Condensa	ate Pump 2 and	l Heater D	rain Pu	mp 2	
SRO	D Refer to DB	-OP-02504, Rap	id Shutdown				
SRO		he power reduct er reduction for 2			level, a	nd	
ATO	 Full Automa At the L rate spe Select t Monitor 	-02504 Attachme tic to reduce power OAD CONTROL ecified by the Co he target RTP as Reactor Power mately the expedience	ver - Panel, set the mmand SRO s determined by to confirm powe	rate of ch y the Com	ange to mand S	the	
ATC		ulating Rod Inse pecification 3.2.		ring the Sł	nutdown	ı per	

Appendix D)		Оре	erator Actio	n		Forr	n E	S-D-2
Op Test No.:		Scenario #			1 and 2		2	of	2
Event Descrip	otion:	D2 Lockout P	ower Re	eduction SAC	1 fails to Auto s	tart			
Time	Position			Applicant	's Actions or Bel	navior			
	ATC	Attempt to negative 20			wer Imbaland PSRs	e betweer	n 0 and	b	
	BOP	Control Co and LIC42		ate Flow at	approximate	ly 7 MPPH	l using	g Ll	C420
	BOP	to remove t	he Au	kiliary Feed	power, notify I Pump Turbir cted by Attac	ne Main St	eam N	/ini	
	Team	Recognize	SAC 1	and EIAC	did not auto s	start			
	SRO	Refer to DE	3-OP-0	2528, Loss	s of Instrumer	nt Air for lo	ss of S	SAC	2
	BOP	Start SAC ²	1						
	SRO		onditio	n A due to	lue to loss of Bus Tie Trans es				
	BOP	Dispatch E	Os to E	Bus D2 and	I SAC 1				
When plan	t is stabilize	ed at 85-90%	powe	er and Dea	erator Levels	s are bein	a con	tro	lled
					to event 3,				

Appendix D		Operator Action	Form ES-D-2			
Op Test No.:	2020	Scenario # <u>5</u> Event # <u>3 and 4</u> P	age <u>1</u> of <u>2</u>			
Event Description:		Power range (NI5) high failure and Place RPS Channel 2	in Manual Bypass (TS)			
Time Po	sition	Applicant's Actions or Behavior				
Indications Ava • Annunciator • Annunciator • Rods insertin	5-1-H, F 5-2-H, F	RPS Ch 2 Trip RPS Hi Flux Trip				
С	rew	Recognize that NI5 has failed high				
S	RO	Implement DB-OP-02505, Nuclear Instrumenta	tion Failures			
Α	ATC	 Place the Rod Control Panel in MANUAL Place the Reactor Demand Hand Auto State 	ition in HAND			
В	SOP	IF proper RCS Tave control by Feedwater is NOT indicated, THEN, place BOTH Feedwater Loop Demands to HAND AND adjust FW Demand to stabilize RCS Tave				
С	rew	Determine which Power Range Nuclear Instrur	nent(s) have failed			
s	RO	 Enter TS 3.3.1, Condition A for function 1, Refer to TS 3.3.16 (only 3 ARTS Channels) 				
S	RO	Notify SM to refer to reactivity management, evand Fleet Update requirements	ent notifications,			
s	RO	 Brief contingency actions for Power Range inst RFR will not actuate TBV Bias for reactor trip will not function Main FW block valves will not auto close 	rument failure			
s	RO	Enter TS 3.4.9 Condition A <u>IF</u> Pressurizer water inches during the transient (Pressurizer Level A in at 226 inches)				

Appendix D		Operator Action	Form ES-D-2
Op Test No.:	2020		2_ of _2
Event Descrip		Power range (NI5) high failure and Place RPS Channel 2 in Ma	inual Bypass (TS)
Time	Position	Applicant's Actions or Behavior	
	SRO	 Directs placing RPS Ch 2 in Manual Bypass 06403, Reactor Protective System (RPS) an Instrumentation (NI) Operating Procedure Directs placing the Power Range Test Modu affected channel in TEST OPERATE 	d Nuclear
	BOP	 Obtain RPS Manual Bypass key and RPS C Rotate the MANUAL-BYPASS KEY SWITCH manual bypass relay in RPS Channel 2 Check MANUAL-BYPASS light is BRIGHT Check SUB-SYSTEM light is DIM 	
	BOP	Place the Power Range Test Module for NI5 in TES	TOPERATE
NOTE: ICS		QUIRED TO BE IN AUTO TO CONTINUE SCENARI	0
	SRO	Directs placing ICS in Auto per DB-OP-06401, Integ System Operating Procedure	rated Control
	ATC	 Depress AUTO on HC NI44, ROD CONTRC Verify Tave at setpoint Depress AUTO on HIC ICS20, REACTOR D 	
	BOP	 If necessary, place Feedwater Loop Demands in au Adjust FEEDWATER DEMAND to place meason the caret Press AUTO on FIC ICS32B(A), FEEDWAT 	asured variable
Onlard			Deceter Triv
On Lead Ev	valuator's d	iscretion, proceed to Event 5, Turbine Vibration –	Reactor Irip

Appendix D		Operator Action	Form ES-D-2	
Op Test No.: Event Descrip		Scenario # <u>5</u> Event # <u>5, 6, 7, 8</u> Page Turbine Vibration - Trip Rx, Overcooling, Overfeed, and Loss of N	1of <u>4</u> IUPs	
Time	Position	Applicant's Actions or Behavior		
	Available: 2-E T-G BEAI	RING VIB HI		
	CREW	Recognize Turbine Vibrations increasing		
	BOP	Refer to DB-OP-02015, Turbine Alarm panel 15 Annu	unciators	
	BOP	Identify Turbine bearing vibration above 12 mils		
	ATC	 Trip the reactor GO TO DB-OP-02000, RPS, SFAS S or SG Tube Rupture. Manually trip the reactor Verify power decreasing in the intermediate ra Manually trip the turbine Verify all Turbine Stop Valves OR Control Val closed. 	ange	
	SRO	Route to DB-OP-02000, Section 3		
	SRO/ATC	 Verify immediate actions Verify Reactor is tripped Verify power decreasing in the intermediate ran Verify Turbine is Tripped Verify all 4 Stop Valves or Control Valves are contr	-	
	SRO	Route to DB OP 02000, Section 4 Supplemental Acti	ons	
	CREW	 Implement any necessary Specific Rules Actions ACTIONS FOR LOSS OF SUBCOOLING MA STEAM GENERATOR CONTROL POWER FOR C1 AND D1 BUSES OR EDG S 		

Appendix D		Operator Action	Form ES-D-2				
Op Test No.: Event Descript		Scenario # <u>5</u> Event # <u>5, 6, 7, 8</u> Page Turbine Vibration - Trip Rx, Overcooling, Overfeed, and Loss of	2 of4 ⁷ MUPs				
Time	Position	Applicant's Actions or Behavior					
	CREW	 Implement any necessary Symptom Mitigation Sect Lack of Adequate Subcooling Margin Lack of Heat Transfer Overcooling Steam Generator Tube Rupture 	tions				
	ATC	Check for all Group 1-7 Control Rods fully inserted					
	ATC	Perform Attachment 1, Primary Inventory Control A	ctions.				
	ATC	 Transfer MU Pump suctions to the BWST pupess OFF for each switch. MU 6405 MU3971 Set Pressurizer Level Controller to 100 inch 					
	ATC	 Recognize MUP 1 tripped Check Component Cooling water is available Close MU19. Close MU32 Attempt to Start the STBY MUP 	e to the RCPs.				
	ATC	 Perform Attachment 8, Place HPI/LPI/MU in start HPI and LPI. Notify the CSRO to reduce RCS pressure us AND spray to 1700 to 1800 psig to allow HF maintain PZR level. 	sing heaters				
	ATC	 Verify both HPI Trains are in service as follows: Verify both CCW Trains are in service Verify both HPI Pumps are running Verify all HPI Injection valves are open (HP2C HP2D HP2A HP2B) 					
	ATC	 Close MU32 Attempt to Start the STBY MUP 1. Perform Attachment 8, Place HPI/LPI/MU in start HPI and LPI. 2. Notify the CSRO to reduce RCS pressure us AND spray to 1700 to 1800 psig to allow HF maintain PZR level. Verify both HPI Trains are in service as follows: Verify both HPI Trains are in service as follows: Verify both HPI Pumps are running Verify all HPI Injection valves are open 	sing heaters				

Appendix D		Operator Action	Form ES-D-2
Op Test No.:	2020	Scenario # <u>5</u> Event # <u>5, 6, 7, 8</u> Page	3 of
Event Descript	tion:	Turbine Vibration - Trip Rx, Overcooling, Overfeed, and Loss of	MUPs
Time	Position	Applicant's Actions or Behavior	
	ATC	 Verify both LPI Trains are in service as follows: Verify both LPI Pumps are running Open both piggyback valves (DH64 DH63) 	
	•	ce in DB-OP-02512, Makeup and Purification Syst tachment 6, RCS Pressure Control After Reactor	
			<u></u>
*Critical Task (CT-30)	ATC	 Maintain Pressurizer Level Turn off all PZR Heaters. *Reduce RCS pressure to between 1700 an using RC 2, PZR SPRAY VALVE. *Throttle HPI flow to maintain Pressurizer le 	
	BOP	Perform Attachment 2: SG Inventory and Pressure Actions	Control
	BOP	Verify Steam Generator Levels are being controlled Rule 4, Steam Generator Control.	per Specific
	BOP	Verify Steam Generator Pressures are being contro automatic operation of the TBVs	olled by
	BOP	Identify Overcooling	
	SRO	Route to Section 7.0 Overcooling	
	ATC	Verify Attachment 8, Place HPI/LPI/MU in service	
	SRO	Implement Attachment 20, Isolate or Control Potent Overcooling	ial Source of
	BOP	 Identify MSR Second Stage Reheat Steam Valves a MS199, MSR 1 REHEAT STEAM SOURCE MS195B, MSR 1 REHEAT STEAM HIGH L 	

Appendix D		Operator Action	Form ES-D-2		
Op Test No.:	2020	Scenario # _5 _ Event # _5, 6, 7, 8 _ Page	e <u>4</u> of <u>4</u>		
Event Descript	ion:	Turbine Vibration - Trip Rx, Overcooling, Overfeed, and Loss	s of MUPs		
Time	e Position Applicant's Actions or Behavior				
	BOP	Attempt to close MSR Second Stage Reheat Ste	am Valves		
	Booth Cue	If called, role play as EO, "MSR Second Stage Valves cannot be closed locally" • MS199, MSR 1 REHEAT STEAM SOUR • MS195B, MSR 1 REHEAT STEAM HIGH	CE		
Critical Task (CT-17)	BOP	Terminate the overcooling. Manually Initiate and	Isolate SFRCS		
	BOP	 Verify proper SFRCS actuation Verify SG pressures are equal Control AVVs to maintain RCS temperature condecreasing Verify proper SG level control 	onstant or slowly		
	BOP	Recognize SG2 above setpoint with full Aux Fee	dwater flow		
	DOF				
Critical Task (CT-16)	BOP	Control SG2 level by performing one or more of t Reduce AFPT 2 speed Close AF599 	he following:		
		l is being maintained in a band of 80-120 inche / SR4, the Scenario can be terminated	s and SG2 level		

Justification for Critical Tasks

The term "Critical Task", is defined in NUREG-1021, OPERATOR LICENSING EXAMINER STANDARDS FOR POWER REACTORS. The listed critical tasks were compiled based on a review of Areva Technical Document 47-1229003-06, EOP Technical Bases Document, and additional CTs were added based on established DB Operations expectations and standards for previously identified Critical Task.

CT-17: Isolate Overcooling SG

- Safety Significance: The MSR Second Stage Reheat Valves fail to auto close which causes an overcooling
- 2. Cues:
 - SG Pressure lowering to less than post trip expected value
 - Turbine Bypass Valves modulate closed
 - DB-OP-02000 Section 7, Overcooling
- 3. Measured by:

Manually I&I SFRCS or align valves per Table 1 DB-OP-02000 to isolate overcooling within \leq 10 minutes of SG(s) pressure \leq 630 psig

- 4. Feedback:
 - SG Pressure
 - Valve indication aligned per Table 1 of DB-OP-02000 such that MSIV of affected SG is closed.

CT-16: FW Flow Control

- 1. Safety Significance: Control FW to mitigate excessive Pri/Sec heat transfer
- 2. Cues:
 - SG2 above level setpoint with full Aux Feedwater flow
 - DB-OP-02000 Specific Rule 4
- 3. Measured by:

FW flow to the affected SG(s) must be controlled within \leq 10 minutes of SG(s) pressure \leq 630 psig

- 4. Feedback:
 - Lowering Aux Feedwater flow to SG 2
 - Reduce AFPT 2 speed or AF599 indicating Closed

Justification for Critical Tasks

The term "Critical Task", is defined in NUREG-1021, OPERATOR LICENSING EXAMINER STANDARDS FOR POWER REACTORS. The listed critical tasks were compiled based on a review of Areva Technical Document 47-1229003-06, EOP Technical Bases Document, and additional CTs were added based on established DB Operations expectations and standards for previously identified Critical Task.

CT-30: Control RCS Inventory

- 1. Safety Significance: Control RCS Pressure, HPI, and Letdown such that Pressurizer Level is being maintained to Prevent Degradation of Mitigative Capability of the Plant
- 2. Cues:
 - Makeup Pump Status
 - DB-OP-02000 Attachments 1, 8 & 13, and Specific Rule 3
- 3. Measured by: Lowering RCS Pressure to allow High Pressure Injection flow into the RCS
- 4. Feedback:
 - Rising Pressurizer level
 - High Pressure Injection flow into the RCS

SIMULATOR SETUP INFORMATION

- 1. Simulator Setup
 - 100% Power
 - 1 and 3 TPCW Pumps in service
 - SFRCS disabled
 - MUP 1 in service

2. Procedures

- DB-OP-02515, Nuclear Instrument Failures Abnormal
- DB-OP-02526, Primary to Secondary Heat Transfer Upset
- DB-OP-06403, Reactor Protective System (RPS) and Nuclear Instrumentation
- DB-OP-06401, Integrated Control System
- DB-OP-02002, Letdown/Makeup Alarm Panel
- DB-OP-02512, Makeup and Purification System Malfunctions
- DB-OP-02504, Rapid Shutdown
- DB-OP-02000, RPS, SFAS, SFRCS Trip or SG Tube Rupture

EPE 104

SHIFT MGR / UNIT/FIELD SUPERVISOR/ SHIFT ENGINEER TURNOVER CHECKLIST

DATE: Today	SHIFT: <u>NIGHTS</u>	MODE: <u>1</u>	POWER: <u>100</u> %
	PROTEC	TED TRAIN: <u>2</u>	
PLANT RISK: <u>GREEN</u>	SECURITY F	RISK: <u>YELLOW</u>	GRID RISK: <u>GREEN</u>
LICENSE REQUIREMENT	<u>S:</u>		

CONTINGENCY PLANS IN EFFECT:

OUTSTANDING OPERABILITY EVALUATIONS:

ALARM STATUS:

AT THE CONTROLS RO / ZONE 3 EO

CONTROL ROOM LEVEL 1 WORK AROUNDS

CONTROL ROOM LEVEL 2 WORK AROUNDS

CTRM DEFICIENCIES

<u>GENERAL STATUS</u> Maintain 100% power Shift Routines

ZONE 3 – LEVEL 1 WORKAROUNDS ZONE 3 – LEVEL 2 WORKAROUNDS

GENERAL NOTES

BALANCE OF PLANT RO / ZONE 1 EO / ZONE 2 EO

GENERAL STATUS

ZONE 1 – LEVEL 1 WORKAROUNDS ZONE 1 – LEVEL 2 WORKAROUNDS

ZONE 2 – LEVEL 1 WORKAROUNDS ZONE 2 – LEVEL 2 WORKAROUNDS

<u>GENERAL NOTES</u> CCW 1 – 85 °F CCW 2 – 85 °F CCW 3 – 110 °F

Command SRO / Field Supervisor / Shift Engineer Input

CHEMISTRY – STATUSIntake Chlorination – Bays 2 & 3Circwater Chlorination – 1 & 3Circwater Dynacool – I/SSodium Bisulfite – I/SMSD Skid \Rightarrow I/SMSR Drains: #1 \Rightarrow Forward#2 \Rightarrow ForwardPolishers I/S – 1,2,3,4

Evolutions in Progress

Follow-up Items

M&TE Issued for Testing- NOP-WM-5002 Compliance:

EPE 104

Page 2 SHIFT MGR / UNIT/FIELD SUPERVISOR/ SHIFT ENGINEER TURNOVER CHECKLIST

Specification	Equipment Affected	Date & Time Entered	Action Required	Issue Owner/ Test Leader	Next Action Due/ Responsible Individual