



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038  
Hope Creek Generating Station

July 12, 1991

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT  
HOPE CREEK GENERATION STATION UNIT 1  
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for June are being forwarded to you with the summary of changes, tests, and experiments for June 1991 pursuant to the requirements of 10CFR50.59(b).

Sincerely yours,

S. J. Hagan  
General Manager -  
Hope Creek Operations

RAR  
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Attachments

C Distribution

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PDR ADOCK 05000354  
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# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-354  
UNIT Hope Creek  
DATE 7/12/91  
COMPLETED BY V. Zabielski  
TELEPHONE (609) 339-3506

MONTH June 1991

## DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

|     |              |
|-----|--------------|
| 1.  | <u>1020</u>  |
| 2.  | <u>1041*</u> |
| 3.  | <u>1041*</u> |
| 4.  | <u>1051</u>  |
| 5.  | <u>1051</u>  |
| 6.  | <u>1051</u>  |
| 7.  | <u>1058</u>  |
| 8.  | <u>1043</u>  |
| 9.  | <u>1033</u>  |
| 10. | <u>1028</u>  |
| 11. | <u>1017</u>  |
| 12. | <u>1067</u>  |
| 13. | <u>948</u>   |
| 14. | <u>1047</u>  |
| 15. | <u>986</u>   |
| 16. | <u>1006</u>  |

## DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

|     |             |
|-----|-------------|
| 17. | <u>1035</u> |
| 18. | <u>1042</u> |
| 19. | <u>997</u>  |
| 20. | <u>994</u>  |
| 21. | <u>1026</u> |
| 22. | <u>1014</u> |
| 23. | <u>1046</u> |
| 24. | <u>1039</u> |
| 25. | <u>1044</u> |
| 26. | <u>1027</u> |
| 27. | <u>1040</u> |
| 28. | <u>943*</u> |
| 29. | <u>942*</u> |
| 30. | <u>1021</u> |
| 31. | <u>N/A</u>  |

\*Due to an error in recording the meter readings, the exact average daily power levels for June 2, 3, 28 and 29 are unknown. The listed averages for those four days represent the average of the appropriate two-day total.

# OPERATING DATA REPORT

DOCKET NO. 50-354  
UNIT Hope Creek  
DATE 7/12/91  
COMPLETED BY V. Zabielski  
TELEPHONE (609) 339-3506

## OPERATING STATUS

1. Reporting Period June 1991 Gross Hours in Report Period 720
2. Currently Authorized Power Level (MWt) 3293  
Max. Depend. Capacity (MWe-Net) 1031  
Design Electrical Rating (MWe-Net) 1067
3. Power Level to which restricted (if any) (MWe-Net) None
4. Reasons for restriction (if any)
5. No. of hours reactor was critical

|                                             | This<br>Month    | Yr To<br>Date    | Cumulative         |
|---------------------------------------------|------------------|------------------|--------------------|
| 5. No. of hours reactor was critical        | <u>720.0</u>     | <u>2962.8</u>    | <u>32,744.3</u>    |
| 6. Reactor reserve shutdown hours           | <u>0.0</u>       | <u>0.0</u>       | <u>0.0</u>         |
| 7. Hours generator on line                  | <u>720.8</u>     | <u>2864.5</u>    | <u>32,157.6</u>    |
| 8. Unit reserve shutdown hours              | <u>0.0</u>       | <u>0.0</u>       | <u>0.0</u>         |
| 9. Gross thermal energy generated (MWH)     | <u>2,352,120</u> | <u>9,124,158</u> | <u>101,666,565</u> |
| 10. Gross electrical energy generated (MWH) | <u>768,440</u>   | <u>3,007,551</u> | <u>33,629,224</u>  |
| 11. Net electrical energy generated (MWH)   | <u>736,723</u>   | <u>2,870,038</u> | <u>32,126,722</u>  |
| 12. Reactor service factor                  | <u>100.0</u>     | <u>68.2</u>      | <u>82.5</u>        |
| 13. Reactor availability factor             | <u>100.0</u>     | <u>68.2</u>      | <u>82.5</u>        |
| 14. Unit service factor                     | <u>100.0</u>     | <u>66.0</u>      | <u>81.0</u>        |
| 15. Unit availability factor                | <u>100.0</u>     | <u>66.0</u>      | <u>81.0</u>        |
| 16. Unit capacity factor (using MDC)        | <u>99.2</u>      | <u>64.1</u>      | <u>78.5</u>        |
| 17. Unit capacity factor (Using Design MWe) | <u>95.9</u>      | <u>61.9</u>      | <u>75.9</u>        |
| 18. Unit forced outage rate                 | <u>0.0</u>       | <u>9.7</u>       | <u>5.9</u>         |
19. Shutdowns scheduled over next 6 months (type, date, & duration):  
None
20. If shutdown at end of report period, estimated date of start-up:  
N/A

OPERATING DATA REPORT  
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-354  
UNIT Hope Creek  
DATE 7/12/91  
COMPLETED BY V. Zabielski  
TELEPHONE (609) 339-3506

MONTH June 1991

| NO. | DATE | TYPE<br>F=FORCED<br>S=SCHEDULED | DURATION<br>(HOURS) | REASON<br>(1) | METHOD OF<br>SHUTTING<br>DOWN THE<br>REACTOR OR<br>REDUCING<br>POWER (2) | CORRECTIVE<br>ACTION/COMMENTS |
|-----|------|---------------------------------|---------------------|---------------|--------------------------------------------------------------------------|-------------------------------|
|     |      |                                 |                     |               |                                                                          | NONE                          |

Summary

# REFUELING INFORMATION

DOCKET NO. 50-354  
UNIT Hope Creek  
DATE 7/12/91  
COMPLETED BY S. Hollingsworth  
TELEPHONE (609) 339-1051

MONTH June 1991

1. Refueling information has changed from last month:  
Yes                      No ☒
2. Scheduled date for next refueling: 9/5/92
3. Scheduled date for restart following refueling: 11/3/92
4. A. Will Technical Specification changes or other license amendments be required?  
Yes                      No ☒

B. Has the reload fuel design been reviewed by the Station Operating Review Committee?  
Yes                      No ☒

If no, when is it scheduled? not currently scheduled
5. Scheduled date(s) for submitting proposed licensing action: N/A
6. Important licensing considerations associated with refueling:  
- Amendment 34 to the Hope Creek Tech Specs allows the cycle specific operating limits to be incorporated into the CORE OPERATING LIMITS REPORT; a submittal is therefore not required.
7. Number of Fuel Assemblies:  

|                                               |            |
|-----------------------------------------------|------------|
| A. Incore                                     | <u>764</u> |
| B. In Spent Fuel Storage (prior to refueling) | <u>496</u> |
| C. In Spent Fuel Storage (after refueling)    | <u>760</u> |
8. Present licensed spent fuel storage capacity: 4006  
Future spent fuel storage capacity: 4006
9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: July 22, 2007

HOPE CREEK GENERATING STATION

MONTHLY OPERATING SUMMARY

JUNE 1991

Hope Creek entered the month of June at approximately 100% power and operated for the entire month without experiencing any shutdowns or reportable power reductions. On June 30, the plant completed its 49th day of continuous power operation.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS  
FOR THE HOPE CREEK GENERATING STATION

JUNE 1991

The following Temporary Modification Request (TMR) has been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The TMR did not create a new safety hazard to the plant nor did it affect the safe shutdown of the reactor. The TMR did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluation determined that no unreviewed safety or environmental questions are involved.

TMR

Description of Temporary Modification Request

91-029

This TMR installed a jumper across the High Bearing Oil Temperature Trip Switch of the 'B' Control Room Chiller. The jumper will permit the chiller to operate with a defective module/thermistor until a replacement part can be installed.

The following Deficiency Reports (DR's) have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The DR's did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. The DR's did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

DR

Description of Deficiency Report

HIC-91-138

The 'C' Diesel Generator Fuel Oil Header Low Pressure Alarm Tee Fitting was damaged while an 18 month preventive maintenance activity was being performed. The tee fitting was replaced with an elbow because no replacement parts were available. The two fittings are identical except that the elbow does not provide for a test connection.

HTE-91-139

This DR addresses a pinhole leak in Service Water pipe on the 'C' Spray Wash Pump Suction Header. A leakage deflection device has been installed to protect the surrounding equipment from potential water damage. The pipe may be used-as-is until an outage of sufficient duration to restore the pipe to the ASME code.