



CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT

RR#1 • BOX 127E • EAST HAMPTON, CT 06424-9341

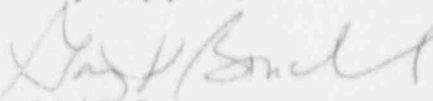
July 15, 1991
Re: Technical Specification 6.9.1.8
Docket No. 50-213

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

In accordance with reporting requirements of Technical Specification 6.9.1.8, the Connecticut Yankee Haddam Neck Plant Monthly Operating Report 06-91 covering operations for the period June 1, 1991 to June 30, 1991 is hereby forwarded.

Very truly yours,


for John P. Steiz
Station Director

JPS/va

- cc: (1) Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406
- (2) John T. Shedlosky
Sr. Resident Inspector
Connecticut Yankee

9107180293 910630
FDR ADOCK 05000213
R FDR

JE24 /

Connecticut Yankee Atomic Power Company

Haddam Neck Plant

Haddam, Connecticut

Monthly Operating Report No. 06-91

For The Month of

June 1991

Plant Operations Summary - June 1991

On June 1st at 0000 hours, the plant was operating at 65% power. At 0032 hours, a power reduction was initiated to align the control rods. At 0115 hours, the plant was at 50% power and the alignment was successfully completed. At 0246 hours, a power ascension was commenced. By 0422 hours, the plant was operating at 65% power.

On June 7th at 0440 hours, the "A" traveling water intake screen was back in service and a power ascension was commenced.

On June 8th at 0550 hours, the plant was operating at 100% power.

On June 20th at 1518 hours, a power reduction was initiated to repair the #1 steam generator feedwater regulating valve. At 1553 hours, the plant was at 95% power to adjust the valve. At 1715 hours, the feedwater regulating valve was returned to service and a power ascension was commenced. At 1845 hours, the plant was operating at 100% power.

The plant continued to operate at 100% power for the remainder of the month.

AVERAGE DAILY UNIT POWER LEVEL

Docket No: 50-213

Unit: Connecticut Yankee
Haddam Neck

Date: July, 15, 1991

Completed By: K. W. Emmons

Month: JUNE 1991

Telephone: (203) 267-3654

<u>DAY</u>	<u>AVERAGE POWER LEVEL</u> (MWe-Net)	<u>DAY</u>	<u>AVERAGE POWER LEVEL</u> (MWe-Net)
1	<u>310</u>	17	<u>558</u>
2	<u>320</u>	18	<u>558</u>
3	<u>321</u>	19	<u>560</u>
4	<u>322</u>	20	<u>554</u>
5	<u>325</u>	21	<u>558</u>
6	<u>334</u>	22	<u>559</u>
7	<u>418</u>	23	<u>559</u>
8	<u>554</u>	24	<u>559</u>
9	<u>563</u>	25	<u>558</u>
10	<u>561</u>	26	<u>556</u>
11	<u>558</u>	27	<u>555</u>
12	<u>557</u>	28	<u>553</u>
13	<u>560</u>	29	<u>551</u>
14	<u>560</u>	30	<u>551</u>
15	<u>558</u>	31	<u>N/A</u>
16	<u>557</u>		

NRC OPERATING STATUS REPORT

Haddam Neck

1. Docket: 50-213
2. Reporting Period: 06/91 Outage + On-line Hours: 0.0 + 720.0 = 720.0
3. Utility Contact: M. P. Bain (203) 267-3635
4. Licensed Thermal Power (MWT): 1825
5. Nameplate Rating (Gross MWe): $667 \times 0.9 = 600.3$
6. Design Electrical Rating (Net MWe): 582
7. Maximum Dependable Capacity (Gross MWe): 591.8
8. Maximum Dependable Capacity (Net MWe): 565
9. If changes occur above since last report, reasons are: NONE
10. Power level to which restricted, if any (Net MWe): N/A
11. Reasons for restriction, if any: N/A

	MONTH	YEAR-TO-DATE	CUMULATIVE
12. Report period hours:	720.0	4,343.0	205,967.0
13. Hours reactor critical:	720.0	4,061.8	165,135.5
14. Reactor reserve shutdown hours:	0.0	0.0	1,285.0
15. Hours generator on-line:	720.0	4,045.2	158,722.8
16. Unit reserve shutdown hours:	0.0	0.0	398.0
17. Gross thermal energy generated (MWtH):	1,204,636.0	7,158,141.0	273,831,154.0 *
18. Gross electrical energy generated (MWeH):	383,344.0	2,380,535.0	89,723,373.0 *
19. Net electrical energy generated (MWeH):	364,002.5	2,269,106.1	85,246,004.7 *
20. Unit service factor:	100.0	93.1	77.1
21. Unit availability factor:	100.0	93.1	77.3
22. Unit capacity factor using MDC net:	89.3	92.5	75.3
23. Unit capacity factor using DER net:	86.9	89.8	71.1
24. Unit forced outage rate:	0.0	6.9	5.9
25. Forced outage hours:	0.0	297.8	10,008.1
26. Shutdowns scheduled over next 6 months: (type,date, duration): 10/19/91 REFUELING - 56 DAY OUTAGE			
27. If currently shutdown, estimated startup date: N/A			

* Cumulative values from the first criticality (07/24/67). (The remaining cumulative values are from the first date of commercial operation, 01/01/68).

MAINTENANCE DEPARTMENT

Report Month: June 1991

System or Component	MALFUNCTION		Effect on Safe Operation	Corrective Action Taken to Prevent Repetition	Special Precautions Taken To Provide For Reactor Safety During Repair
	Cause	Result			
There were no reportable items for Maintenance for the month of June 1991					

I&C DEPARTMENT
Report Month: June 1991

System or Component	MALFUNCTION		Effect on Safe Operation	Corrective Action Taken to Prevent Repetition	Special Precautions Taken To Provide For Reactor Safety During Repair
	Cause	Result			
#1 Steam Flow Transmitter	Dirty Test Switch	Put #1 Feed Flow Channel In Manual Due To Erratic Steam Flow	N/A	Cleaned Test Switch	Put Channel In Manual

CONNECTICUT YANKEE
 REACTOR COOLANT DATA
 MONTH: JUNE 1991

REACTOR COOLANT ANALYSIS	MINIMUM	AVERAGE	MAXIMUM
PH @ 25 DEGREES C	6.49E+00	6.71E+00	6.87E+00
CONDUCTIVITY (UMHOS/CM)	9.95E+00	1.23E+01	1.49E+01
CHLORIDES (PPM)	<5.00E-02	<5.00E-02	<5.00E-02
DISSOLVED OXYGEN (PPB)	<5.00E+00	<5.00E+00	<5.00E+00
BORON (PPM)	2.95E+02	3.77E+02	5.39E+02
LITHIUM (PPM)	8.00E-01	9.00E-01	1.09E+00
TOTAL GAMMA ACT. (UC/ML)	3.85E-01	1.17E+00	2.26E+00
IODINE-131 ACT. (UC/ML)	3.96E-03	5.22E-03	6.97E-03
I-131/I-133 RATIO	1.10E+00	1.45E+00	2.56E+00
CRUD (MG/LITER)	<1.00E-02	<1.00E-02	<1.00E-02
TRITIUM (UC/ML)	1.50E+00	2.11E+00	2.46E+00
HYDROGEN (CC/KG)	1.95E+01	2.62E+01	3.13E+01

AERATED LIQUID WASTE PROCESSED(GALLONS): 1.54E+05
 WASTE LIQUID PROCESSED THROUGH BORON RECOVERY(GALLONS): 1.40E+04
 AVERAGE PRIMARY LEAK RATE(GALLONS PER MINUTE): 3.49E-01
 PRIMARY TO SECONDARY LEAK RATE(GALLONS PER MINUTE): 4.14E-03

UNIT SHUTDOWNS AND POWER REDUCTION

Report Month: JUNE 1991

Docket No: 50-213
 Unit Name: Connecticut Yankee
 Date: July 15, 1991
 Completed By: K. W. Emmons
 Telephone: (203) 267-3654

No.	Date	Type	Duration (Hours)	Reason	Method of Shutting down Reactor	LER Report #	System Code	Component Code	Cause and Corrective Action to Prevent Recurrence
91-03	6/1/91	S	0	A	5	N/A	NN	SCN	Reduced load to enable rebuild of the "A" Circulating Water Traveling water screen due component wear/age. Component was removed from screenwell bay and fully rebuilt.

TYPE

F Forced
 S Scheduled

REASON

A Equipment Failure
 B Maintenance or Test
 C Refueling
 D Regulatory Restriction
 E Operator Training
 F Administrative
 G Operator Error
 H Other (Explain)

METHOD

1 Manual
 2 Manual Scram
 3 Automatic Scram
 4 Continued
 5 Reduced Load
 9 Other

SYSTEM & COMPONENT

Exhibit F & H - Instructions for Preparation of
 Data Entry Sheets
 Licensee Event Report (LER)
 File (NUREG-0161)

Refueling Information Request

1. Name of facility
Haddam Neck
2. Scheduled date for next refueling shutdown.
October 19, 1991
3. Scheduled date for restart following refueling.
December 14, 1991
4. (a) Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
Yes
- (b) If answer is yes, what, in general, will these be?
Revise Section 5 of Technical Specifications to allow use of zircaloy clad fuel. Obtain an exemption from 10CFR50 Appendix K Sections I.D.3, I.D.4 and I.D.5.
- (c) If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload?
(Ref. 10 CFR section 50.59)
n/a
- (d) If no such review has taken place, when is it scheduled?
n/a
5. Scheduled date(s) for submitting proposed licensing action and supporting information.
The exemption request was submitted to the NRC in September 1990. The request for license amendment of Section 5 was submitted in March 1991. The remaining Technical Specification changes were submitted in June 1991.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
Conversion to zircaloy cladding.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.
(a) 157 (b) 709
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
1168
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
1996